



HAL
open science

Creep properties of 9Cr and 14Cr ODS tubes tested by inner gas pressure

Denis L Sornin, Ulla Ehrnstén, Nathanaël Mozzani, Juhani Rantala, Mario Walter, Alexander Hobt, Jarir Aktaa, Elvira Oñorbe, Mercedes Hernandez-Mayoral, Andreas Ulbricht, et al.

► To cite this version:

Denis L Sornin, Ulla Ehrnstén, Nathanaël Mozzani, Juhani Rantala, Mario Walter, et al.. Creep properties of 9Cr and 14Cr ODS tubes tested by inner gas pressure. Metallurgical and Materials Transactions A, 2021, <https://doi.org/10.1007/s11661-021-06327-0>. 10.1007/s11661-021-06327-0 . cea-03263217

HAL Id: cea-03263217

<https://cea.hal.science/cea-03263217>

Submitted on 17 Jun 2021

HAL is a multi-disciplinary open access archive for the deposit and dissemination of scientific research documents, whether they are published or not. The documents may come from teaching and research institutions in France or abroad, or from public or private research centers.

L'archive ouverte pluridisciplinaire **HAL**, est destinée au dépôt et à la diffusion de documents scientifiques de niveau recherche, publiés ou non, émanant des établissements d'enseignement et de recherche français ou étrangers, des laboratoires publics ou privés.

Creep properties of 9Cr and 14Cr ODS tubes tested by inner gas pressure

D. Sornin^{a*}, U. Ehrnstén^b, N. Mozzani^c, J. Rantala^b, M. Walter^d, A. Hobt^e, J. Aktaa^d, E. Oñorbe^f, M. Hernandez-Mayoral^f, A. Ulbricht^g, S. Gicquel^c, L. Frank^h, Y. de Carlan^a

^a Université paris-Saclay, CEA, Service de Recherches Métallurgiques Appliquées, 91191, Gif-Sur-Yvette, France

^b Technical Research Centre of Finland Ltd, Vuorimiehentie 3, FI-02044 VTT, Espoo, Finland

^c EDF Lab Les Renardières, Mechanics and Materials of Components Department, 77250, Ecuelles, France

^d Institut für Angewandte Materialien, Karlsruher Institut für Technologie, KIT, Hermann-von-Helmholtz-Platz 1, 76344 Eggenstein-Leopoldshafen, Germany

^e Materialprüfungsanstalt (MPA) Stuttgart. Actual address: FORM+TEST Seidner&Co. GmbH. Zwiefalter Str. 20. D-88499 Riedlingen

^f CIEMAT, Division of Materials of Energy Interest. Avenida Complutense, 40; 28040-Madrid, Spain

^g Helmholtz-Zentrum Dresden-Rossendorf (HZDR), Bautzner Landstr. 400, 01328 Dresden, Germany

^h Materialprüfungsanstalt (MPA) Universität Stuttgart, Pfaffenwaldring 32, 70569 Stuttgart, Germany

* Corresponding author: denis.sornin@cea.fr

Author Name	ORCID	Email address
Denis L. Sornin	https://orcid.org/0000-0002-9745-0299	denis.sornin@cea.fr
Ulla Ehrnstén	https://orcid.org/0000-0002-3573-1749	Ulla.Ehrnsten@vtt.fi
Nathanaël Mozzani	https://orcid.org/0000-0003-2739-6415	nathanael.mozzani@edf.fr
Juhani Rantala	https://orcid.org/0000-0001-8170-4365	Juhani.Rantala@vtt.fi
Mario Walter		mario.walter@kit.edu
Alexander Hobt	https://orcid.org/0000-0002-6022-8459	hobt@formtest.de
Jarir Aktaa		Jarir.aktaa@kit.edu
Elvira Oñorbe	https://orcid.org/0000-0002-6080-2844	Elvira.Onorbe@ciemat.es
Mercedes Hernandez-Mayoral	https://orcid.org/0000-0003-4504-7577	m.mayoral@ciemat.es
Andreas Ulbricht		a.ulbricht@hzdr.de
Solene Gicquel-Petit	https://orcid.org/0000-0002-1200-8988	solene.gicquel@edf.fr
Lukas Frank	https://orcid.org/0000-0002-4906-0173	Lukas.Frank@mpa.uni-stuttgart.de
Yann de Carlan	https://orcid.org/0000-0001-8522-9004	yann.decarlan@cea.fr

Highlights:

- 9Cr and 14Cr ODS steel tubes are creep tested by inner gas pressure representative of the fission gas loading encountered in service conditions.
- Creeps tests handled by 4 different European teams with different experimental set-up are consistent showing a very low scattering
- The ODS steel tubes typical failure mode is leakage without burst on a longitudinal crack occurring at very low hoop strain.

27 **Key words:** ODS steel, Tube creep test, Inner gas pressure

28

29 **Abstract:**

30 Oxide-dispersion strengthened steels are promising materials for extreme service conditions
31 including nuclear reactors core. In service conditions, nuclear fuel claddings are exposed to the fission
32 gas pressure at temperatures about 700°C. This paper presents novel results on ODS creep properties
33 from a round robin of inner gas pressure creep test. A gas pressure creep test, simulating fission gas
34 loading, was designed and achieved by four different European teams. Lifetime and specific behavior
35 of ODS steel tube are prospected. Based on a mechanical clamping achieving gas tightness, short
36 length tubes samples are tested by different laboratories. In-situ laser measurements exhibits the radial
37 expansion of ODS steel tubes before failure. Post mortem, geometrical characterizations are
38 performed to determine hoop strains at failure. A consistent creep lifetime is observed by all the teams
39 even with slightly different testing apparatus and clamping systems. Under inner gas pressure, ODS
40 steels exhibits a typical failure by leakage associated to a very small radial expansion. This behavior
41 results from a brutal failure (burst) without evidence of tertiary creep stage. This failure mode of ODS
42 cladding in creep conditions is consistently observed on all samples of the study. Inner gas pressure
43 creep tests were compared, for the first time, by four European laboratories on ODS steel tube. This
44 technique, simulating the fission gas pressure loading, is applied on small and mechanically clamped
45 samples. This technique shows a remarkable consistency between the different laboratories results
46 and demonstrates to be efficient for ODS steel cladding tube qualification. The results show a
47 correlation between the creep properties and the microstructure.

48

49 **1. Introduction**

50 Considering technical and economic issues, the coming generation of Sodium Fast nuclear Reactors
51 (SFR) must achieve very high fuel burn-ups. Therefore, the choice of materials is crucial to guarantee
52 safety and reliability of the core components. Most of the structural components of the core are made
53 of stainless steel to provide corrosion resistance in both service and fuel reprocessing conditions [1,
54 2]. As to prevent geometrical distortion at high burn-up, the ferritic grades exhibiting a very low void
55 swelling under irradiation are, however, preferred [3-5]. Within this framework, ODS ferritic steels
56 are foreseen for the fuel cladding material [6-8]. The very fine and stable nanoprecipitation of oxides
57 hinders the dislocation mobility at high temperature which results in the good creep properties of the
58 ODS steels [9], [10]. In service conditions, the tubes must withstand the fission gases pressure at high
59 temperature (about 700°C). At the same time the cladding material is submitted to a neutron
60 irradiation up to 150 dpa. The assessment of creep properties was first made using tensile samples
61 made from bars or a massive block [11, 12]. However, the ODS steel microstructures are known to
62 be very anisotropic, which depends on the used manufacturing and forming processes [13]. Thus,
63 creep results are very dependent on both the microstructure and testing direction [14],[15].

64 Since 2012, CEA manufactures cladding tubes from Hot Extruded rough bars [16], further cold
65 formed by tube pilgering [17]. To assess the behavior of such cladding tubes with a complex
66 microstructure inherited from the manufacturing sequence, the need of advanced creep test emerge.
67 Such tests are expected to be demonstrative of the real loading path during service [18]. However, to
68 perform creep tests on tubular materials, the tube specimen must be plugged to retain the pressure
69 inside the tube, which is not a trivial challenge. Therefore, within the framework of the European EU-
70 MatISSE project, a task was dedicated to inner pressure creep tests on ODS cladding tubes involving
71 four Europeans teams which developed their own plugging system and test apparatus and performed
72 the tests.

73 Welding is the most typical technique used for plugging conventional materials for internal pressure
74 creep tests. However, ODS-materials lose their creep strength when melted, which must be taken into
75 account when designing the test set-up. Rather little openly available information on ODS tube
76 material internal pressure tests and plugging techniques was found in literature [18], [15]. Different
77 methods for plugging have been prospected in earlier study [19]. Pressurized resistance welding
78 (PRW) was often selected over uniaxial diffusion welding, spark plasma sintering or friction stir
79 welding [18] [20]. Magnetic pulse welding [21] and friction stir welding [22] have also been
80 developed for welding fuel bundle plugs.

81 For example, JAEA has developed tubes creep tests using two end plugs welded by PRW [18]. This
82 technics enables a large number of tests in the same furnace [23], [24] but is not optimal considering
83 the deformation monitoring during testing. At CEA, electron beam welding is used for manufacturing
84 of long samples where welded regions are protected during testing by water boxes [15]. This last
85 technique is expensive and material consuming due to the complexity of the sample apparatus.

86 In the present work, a mechanical plug was developed and used. A mechanical plug allows more
87 efficient use of the material, as the HAZ of the welds does not need to be protected during the test,
88 reducing the overall tube length. Moreover, for a better understanding of the failure mode of ODS
89 tubes, diametrical expansion monitoring during testing is useful and possible. , VTT has successfully
90 developed a mechanical plug for internal pressure testing of 15/15Ti steel tubing, which indicated
91 that this type of plugging can also be used for ODS tubes [25]. The plugging was slightly modified
92 for ODS materials. While KIT and MPA Stuttgart used slightly modified types of VTT's plugging
93 system, EDF developed their own experimental system based on mechanical plugs.

94 In this study not only lifetime but also creep deformation were captured showing the singular failure
95 mode of ODS steels. Post-mortem analysis were also carried out to identify the crack geometry.

96 This study evidence the creep behavior of ODS tubes under temperatures and loading path similar to
97 the service conditions.

98
99

100 **2. Materials and methods**

101 2.1 Material manufacturing

102

103 ODS steels are the results of a complex manufacturing sequence involving powder metallurgy. Pre-
104 alloyed gas-atomized metallic powder is used with a D50 of about 80 μm in two different Cr contents,
105 9 and 14 wt%. These powders are then mechanically alloyed, under H_2 atmosphere, with 0.25 wt%
106 of Y_2O_3 and 0.3 wt% TiH_2 . The nominal composition of the various grades of the study is presented
107 in Table 1. The powders are processed in one or two Hot Extrusion batches. The powder is put in soft
108 steel cans of about 3 kg and air vacuumed during 4 h at 400°C. After sealing, the billets are heated
109 for 1 h at 1100°C in a radiative furnace. During this treatment, nano-oxides precipitate [26],[27].
110 Then the billets are Hot Extruded (HE) using a circular die and a 675 ton hydraulic press with an
111 extrusion ratio of $R=12.5$. After removing the cans and deep drilling, seamless rough tubes are
112 obtained with a diameter about $\varnothing 16$ mm. Before cold pilgering, a specific heat treatment is applied
113 depending on the composition of the tubes. For the ferritic grades (14Cr), a 3 h heat treatment at
114 1200°C releases stress induced by hot extrusion. For the ferritic-martensitic grade (9Cr), a softer
115 initial treatment of 1h at 1050°C is imposed. Then, a sequence of four cold working passes by High
116 Precision Tube Rolling (HPTR) pilger mill is applied with intermediate heat treatment at 1200°C and
117 1050°C for 14Cr and Cr respectively. The cumulated logarithmic strain is about 40% for each of the
118 4 passes. The final dimensions are $\varnothing 10.73 \times 0.5$ mm and a final heat treatment is applied depending on

119 the grade. For 14Cr ODS grade, a 750°C/1 h recovery treatment is applied after the last pass whereas
 120 for the 9Cr ODS grade, quenching is applied resulting in a martensitic structure tempered at 750°C/30
 121 min. Then, tubes are straightened and optically controlled. About 2000 mm of each grade was
 122 produced and dispatched to the EU-MaTISSE project partners.

123
 124

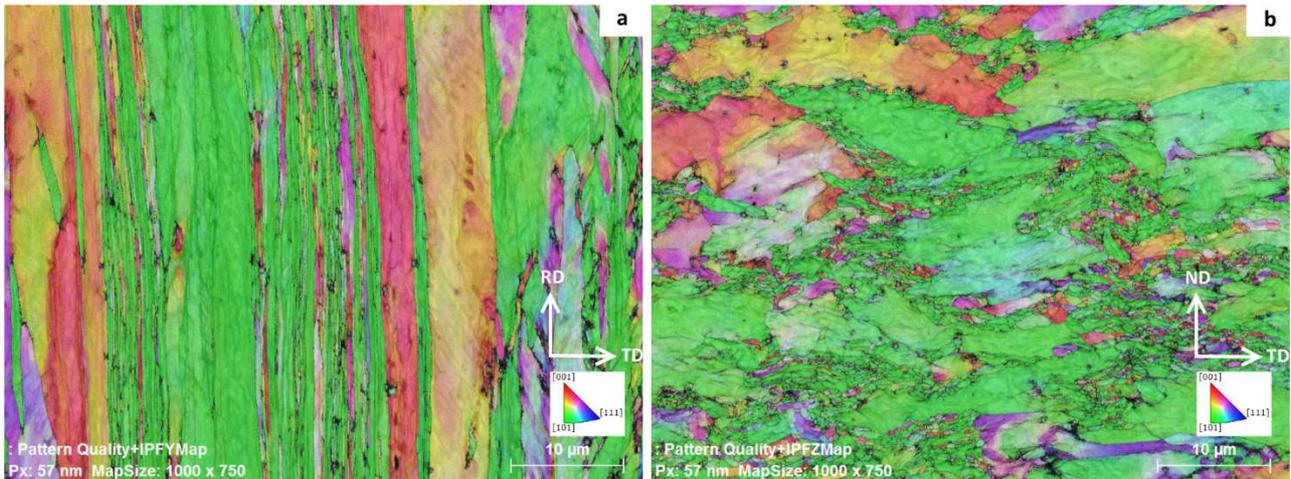
Table 1. Nominal composition of the ODS steel grades.

Grade	Nominal composition (wt-%)							Final heat treatment
	Cr	W	C	Y ₂ O ₃	Ti	Ni	Fe	
9Cr ODS	9	1	0.1	0.25	0.2	0.15	Bal.	Quenched + tempered 760°C/30min
14Cr ODS	14	1	0	0.25	0.3	0	Bal.	750°C/1h

125
 126
 127

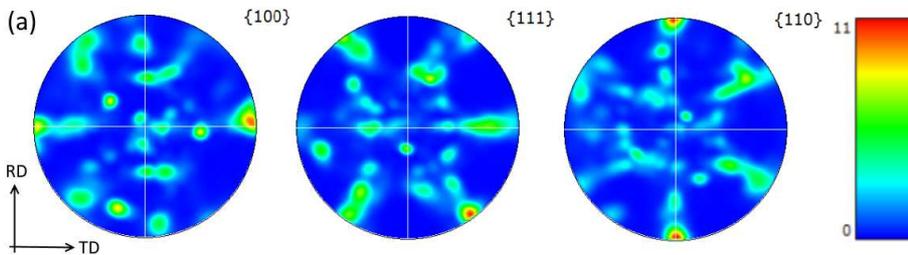
2.2 Microstructural investigations

128 SEM EBSD orientation maps are presented in Figure 1 and Figure 3 for the 14Cr and 9Cr ODS tubes,
 129 respectively. The inverse pole figures colour code is plotted respectively to the longitudinal direction
 130 (RD). ND corresponds to the radial direction of the tube. For both tubes, poles figures corresponding
 131 to the {100}, {111} and {110} planes are plotted in Figure 2 and Figure 4.

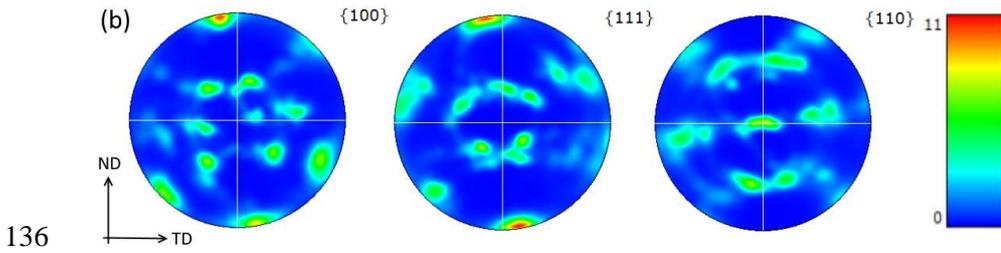


132
 133
 134

Figure 1. SEM EBSD orientation maps along RD for the 14Cr ODS tube, (a) in longitudinal and (b) transversal direction (CEA).



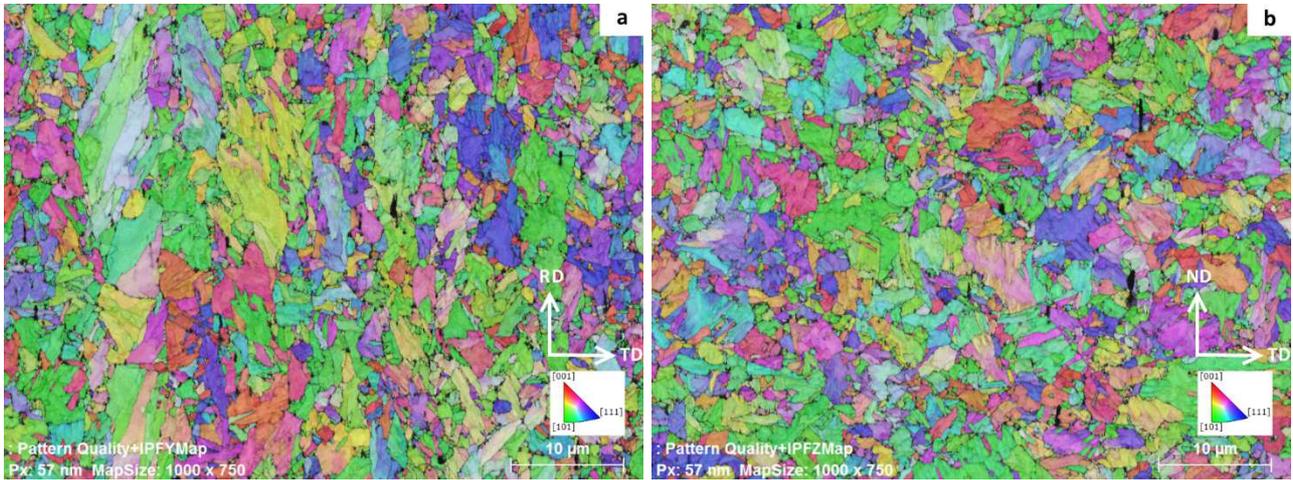
135



137 *Figure 2. {100}, {111} and {110} poles figures computed from EBSD maps for the 14Cr ODS tubes projected on, (a) RD-TD and (b)*
 138 *ND-TD planes (CEA).*

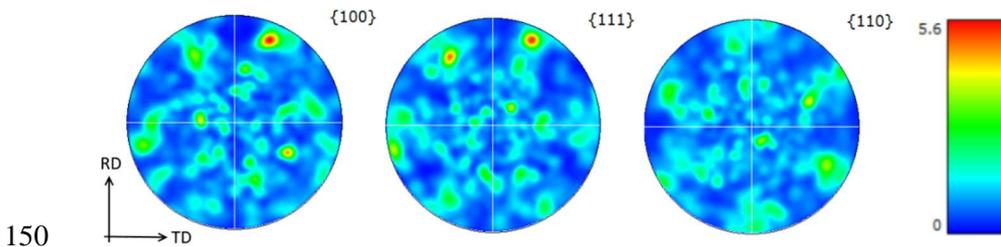
139 For the fully ferritic 14Cr ODS tube material, Figure 1 show clearly an anisotropic microstructure.
 140 The ferritic grains are elongated in the axial orientation with a size of tens of micrometres, while the
 141 size in the opposite direction is a few micrometres. Due to the reduced number of grain captured by
 142 the map, Figure 2 is more indicative than quantitative. However, the well-known rolling texture, α
 143 $\langle 110 \rangle$ fiber along rolling direction is observed for the ferritic 14Cr ODS tube. Consequently, 14Cr
 144 ODS material is assumed to exhibit strong creep properties anisotropy.

145



146 *Figure 3. SEM EBSD orientation maps along RD for the 9Cr ODS (K30-M3) tube, (a) in longitudinal and (b) transversal direction*
 147 *(CEA).*

149



151 *Figure 4. {100}, {111} and {110} poles figures computed from EBSD maps for the 9Cr ODS tubes projected on RD-TD plan (CEA).*

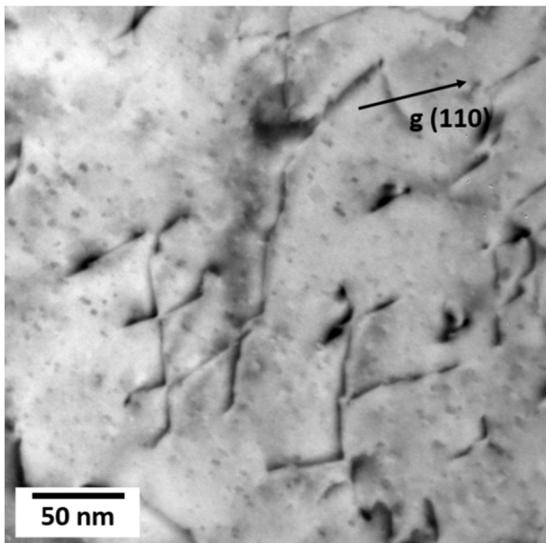
152 The microstructure of the tempered martensite 9Cr ODS tube material is predominately constituted
 153 of equiaxed packets of few microns in size. The martensitic structure is very specific to ODS grades
 154 with less than 11 wt% Cr. It comes from previous austenitic grains that are only a few microns in
 155 size. This leads to a transformation in which one variant per previous martensitic grain is often

156 observed. To the end, at the macroscopic scale, a rather isotropic structure is obtained. It is formed of
157 small packets of ferrite laths, very entangled, which are difficult to discern by SEM.

158
159 A weak α fiber is noticed in Figure 4 compared to 14Cr ODS. Contrary to the Japanese 11Cr ODS
160 grade [28], the material is entirely constituted of martensitic domains excluding residual ferrite.

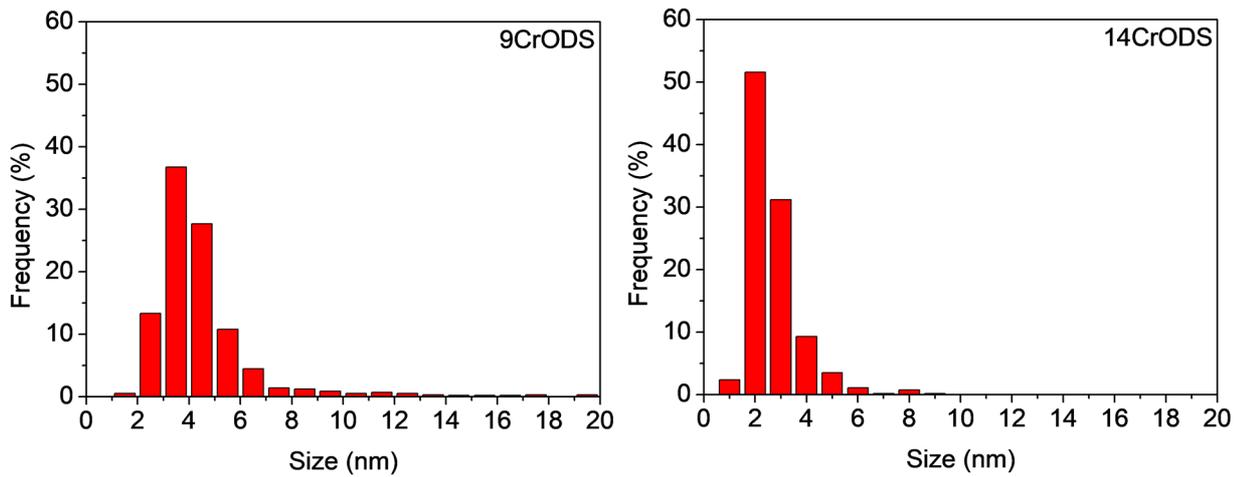
161 Specimens for TEM observations were prepared by electrolytic polishing of 3 mm discs drilled from
162 the tube wall, using the reactive mixture of 5% perchloric acid and methanol at $-60\text{ }^{\circ}\text{C}$. Usual imaging
163 methods such as Bright Field (BF) and Weak Beam Dark Field (WBDF) were employed in a TEM
164 JEOL JEM-2010 with LaB_6 filament operated at 200 keV. Size distribution was estimated by image
165 analysis technique measuring the longer axis of precipitates, using at least five different areas for each
166 measurement employing JMicroVision software [29]. The microstructure of the 9Cr ODS tube is also
167 described with SEM by E. Oñorbe et al., .The same grain structure with large precipitates (mainly
168 TiO_2 about 100 nm in diameter) within grains and at grain boundaries was also visible [30]. Dense
169 networks of dislocations were present within grains and a homogeneous distribution of nano-oxides
170 where dislocations were pinned. The microstructure of 14Cr ODS tube consisted in large columnar
171 ferritic grains with a high dislocation density after rolling process. Figure 5 shows an example of the
172 fine distribution of nano-oxides in the 14Cr ODS. Figure 6 shows the size distribution obtained for
173 both materials. Precipitates appear smaller in 14Cr ODS grade compared to the 9Cr martensitic ODS
174 steel with an average diameter (2.4 ± 0.9) nm in 14Cr and (4 ± 1) nm in 9Cr ODS.

175



176 *Figure 5. TEM BF image of the dispersion of nano-oxides in 14Cr ODS tube (CIEMAT).*

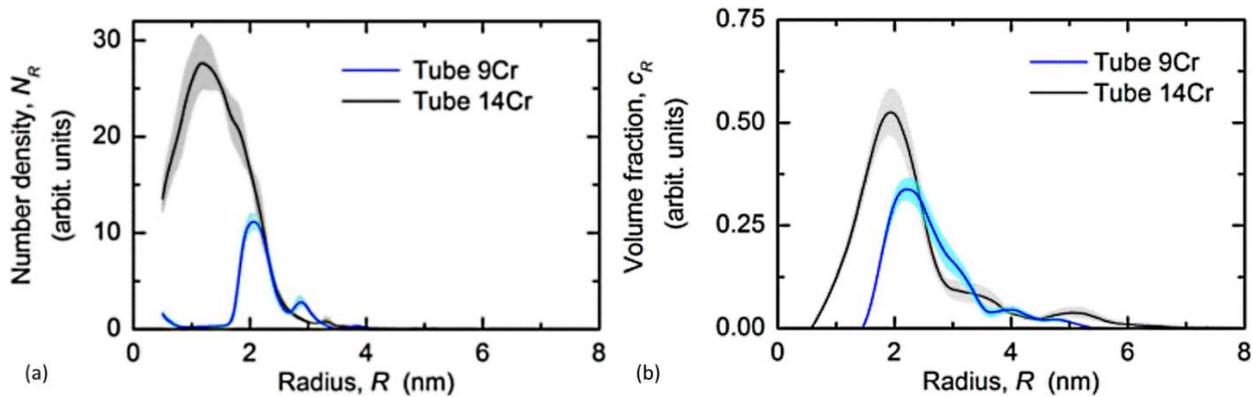
177



178

179 *Figure 6: Size distribution (diameter) of nano-oxides as measured by TEM in 9Cr ODS and 14Cr ODS tubes (CIEMAT).*

180



181

182 *Figure 7. SANS characterization, (a) size distributions in terms of number density of scatters per size increment, (b) size distributions*
 183 *in terms of volume fraction of scatters per size increment (HZDR).*

184 Precipitate distribution was also determined using Small Angle Neutron Scattering (SANS) for both
 185 14Cr and 9Cr ODS grades on D22 at ILL Grenoble [31]. In Figure 7, the maximum density is noticed
 186 at 1.5 nm for 14Cr ODS and about 2 nm for 9Cr ODS. Density appears slightly higher in 14Cr
 187 compared to 9Cr ODS. Both SANS and TEM characterizations consistently exhibit a fine
 188 precipitation in both materials with a higher density and reduced size in 14Cr compared to 9Cr ODS.

189

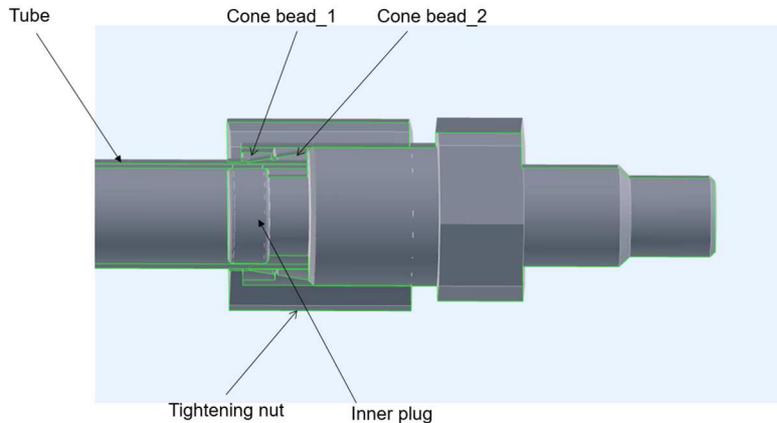
190 **2.3 Creep testing**

191 Internal pressure creep tests were performed by four laboratories using tube specimens with 45 mm
 192 length (one laboratory used 90 mm long specimens). A short specimen length was mainly chosen
 193 based on the amount of tube length available for the tests. Before the tests were started, the plugging
 194 of the tubes using mechanical plugging had to be developed and verified. All laboratories were
 195 successful in this task. The internal pressure tests were performed at 700°C and pressures between 90
 196 and 120 bar for the 9Cr ODS tube material, and at both 650° and 700°C and pressures between 100
 197 and 181 bar for the 14Cr ODS tube material

198 **2.3.1 Creep testing at VTT**

199 The principle of the mechanical plugging at VTT is based on two conical clamping rings compressed
 200 against each other as shown in Figure 8. The Multi Physics software was used in the selection of plug
 201 material and plug structure. Stress analyses were performed using Iron Cad software with a Multi

202 physics program. According to these calculations, the best materials for both the plug beads and rings
203 is 304 stainless steel. Compressive stresses were confirmed at the plug, and the structure did not cause
204 any stress localisation in the tube close to the plug.



205

206 *Figure 8. Drawing showing the plugging design by VTT.*

207 Two thermocouples were installed in the middle of the specimen gauge section, but not tied onto to
208 the tube to avoid scratching. ODS-materials are known to be very notch sensitive, and scratches could
209 easily affect the results. The tube specimens were pressured using argon gas, while the environment
210 on the outer surface was air. The tests were mainly continued until failure. In a few cases, the tests
211 were interrupted without failure after 1800 hours. The last test was performed using biaxial loading
212 with axial/hoop stress ratio of 1. The main intention of this was to demonstrate the effectiveness of
213 the plugging in this type of loading for future use. Biaxial testing is useful to produce data for
214 improved models incorporating the material anisotropy, but also to produce data for different loading
215 scenarios during use, including dry storage, where biaxial loading may prevail. The location of the
216 crack was determined after the test to evaluate a possible influence of the plugging on the results.
217 Most specimens were also subjected to dimensional measurements and scanning electron microscopy
218 (SEM).

219 **2.3.2 Creep testing at EDF**

220 The experimental setup, used by EDF for the ODS cladding tubes, was originally designed for testing
221 Zirconium alloy claddings for pressurized water reactors (PWR). Argon is used as the pressurizing
222 gas. The furnace was modified at the beginning of the EU-MatISSE project in order to reach higher
223 temperatures (up to 800°C) and to accommodate to tube burst (i.e., adding an inner shield to the
224 furnace). Several creep-to-rupture tests were performed on Zircaloy cladding and were successful.
225 Temperature homogeneity was checked at 800°C, and an 8°C gradient was found on 120 mm long
226 specimens. Maximum temperature was measured at the middle of the specimen and the lowest near
227 the bottom plug. Such temperature variations will ensure that failure will occur near the middle of the
228 specimen, as was effectively seen in the tests.

229

230 During the test, the cladding deformation is monitored at mid-length by a laser with an accuracy of
231 5 µm (<0.1 %). The maximum pressure available is 500 bar. Three thermocouples are taped to the
232 specimen to control the furnace. Before and after testing, dimensional controls are performed on the
233 specimen. Based on the experience achieved from PWR cladding tube testing, the chosen plugging
234 technique is inspired by the Swagelok tube fitting principle, but several improvements were

235 undertaken for ODS cladding tubes. The use of an inner filling part on which the tube will collapse
236 was introduced and the different materials were changed to take into account the high mechanical
237 strength of ODS tubes and the need to reuse the pressure fittings. Since the available tube-length is
238 small, only 90 mm long specimens were used, instead of 120 mm for Zircaloy tubes. The first high
239 temperature tests were performed using another ODS test tube provided by CEA. These proved the
240 homogeneity of temperature (± 1 % along the specimen) and the tightness of the circuit.

241

242 **2.3.3 Creep testing at KIT**

243 At KIT, a universal INSTRON (Type 4505) testing machine, equipped with a 5-zone radiation
244 furnace, was used. To apply a controlled internal pressure using argon during testing, a new pressure
245 unit was built, allowing pressurizing a sample between 10 and 200 bar with a deviation of less than
246 0.3 bar related to a set value. After completion, the entire test facility was successfully validated by
247 different specified tests on welded Zircaloy 4 specimens. Additionally, one test on an ODS test tube
248 sample was performed in order to ensure the usability of slightly modified mechanical clamps,
249 originally developed at VTT.

250 For post-test investigations, a customized laser measurement device from ANT Antriebstechnik
251 GmbH (Germany) with a measurement accuracy of 1 μm was used to generate 3D scans of the tested
252 samples to determine the shape evolution depending on the test conditions. The measurements were
253 conducted such, that the diameter of a sample was determined over the entire length for a multitude
254 of positions, using a step size of 500 μm between two adjacent positions. In order to examine the
255 roundness of a specimen, a sample was rotated degree-wise at every longitudinal position and 360
256 values per plane were measured (generally, it would be enough to measure up to a rotation of 180°
257 since afterwards the measurement is more or less repeated). Afterwards, the diameter at every
258 longitudinal position was calculated by averaging the 360 measured values and finally, these averaged
259 values were used to calculate the related hoop strains at the outer surface along a sample.

260

261

262 **2.3.4 Creep testing at MPA**

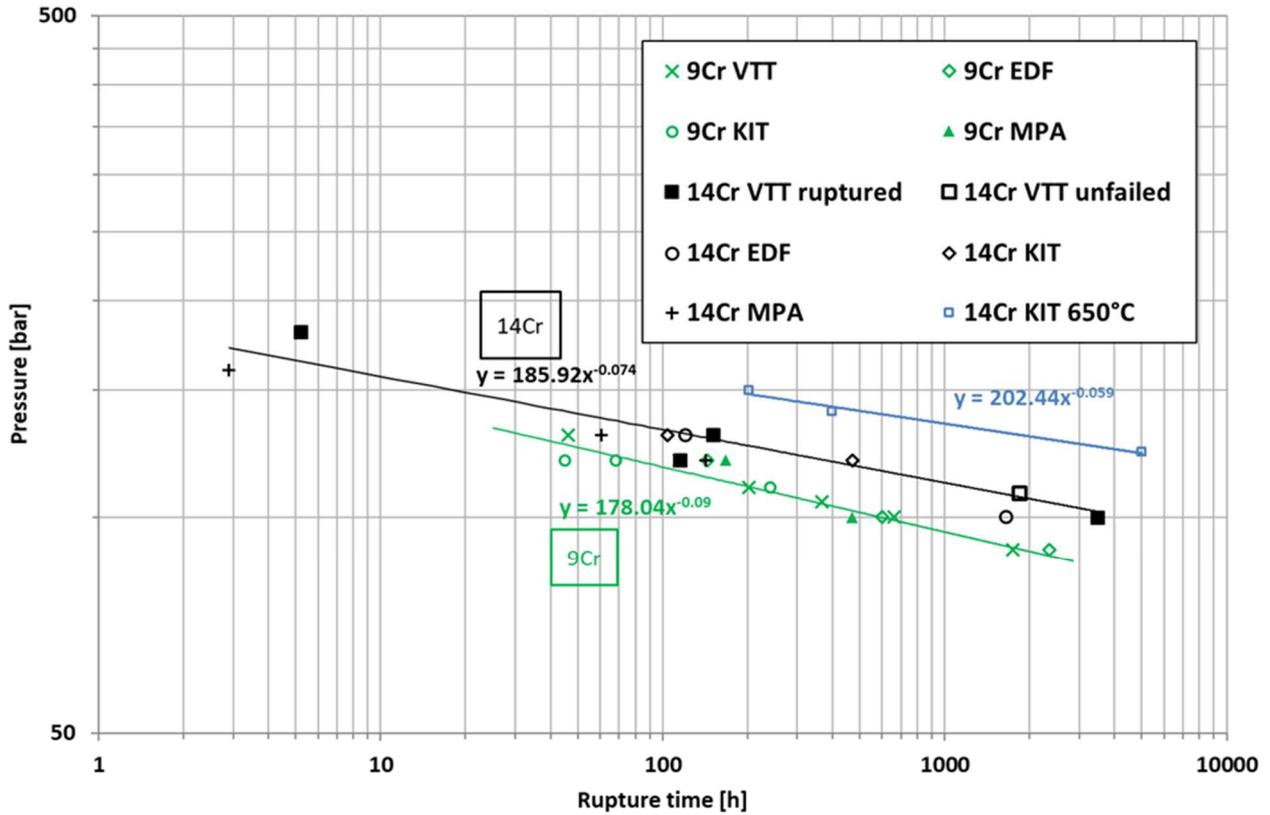
263 At MPA Stuttgart, a plugging type, modified based on the VTT design, was applied. The tube was
264 clamped between conical elements, allowing a minimum influence on the notch sensitive material.
265 The specimen was tested in a 4 column servo-hydraulic universal test machine with a new control
266 electronics. The specimen was heated using a 3-zone radiation furnace. Three thermocouples were
267 applied to measure the temperature along the load train: one couple on each specimen grip and one
268 on the centre of the specimen. The temperature gradient could be kept below 1°C and is in accordance
269 to the given requirements for standard creep tests. The pressure was applied with compressed argon
270 gas. During the creep tests, axial load was zero. This means that only the effect of the inner pressure
271 is accommodated within the tube itself. The pressure was applied after heating up of the system and
272 kept constant within 2% of accuracy. The pressure inlet had a certain length inside the heated area to
273 avoid re-cooling of the tube when applied the pressure and gas flow into the tube. The axial strain
274 was measured during the test using a high temperature contacting extensometer. The contact pressure
275 of the extensometer was minimised since the final failure crack was at a different position than the
276 contact points. The length and the diameter were measured both before and after the test with a precise
277 light projector which is capable of measuring small dimensions. It is commonly used at the MPA to
278 measure creep strain.

279

280 **3. Results**

281 **3.1 Creep properties**

282 The results from the internal pressure tests are presented in Figure 9. The behavior of the tube
 283 materials is very consistent, with smaller scatter than what is usually observed in this type of results
 284 even for conventional industrial materials.



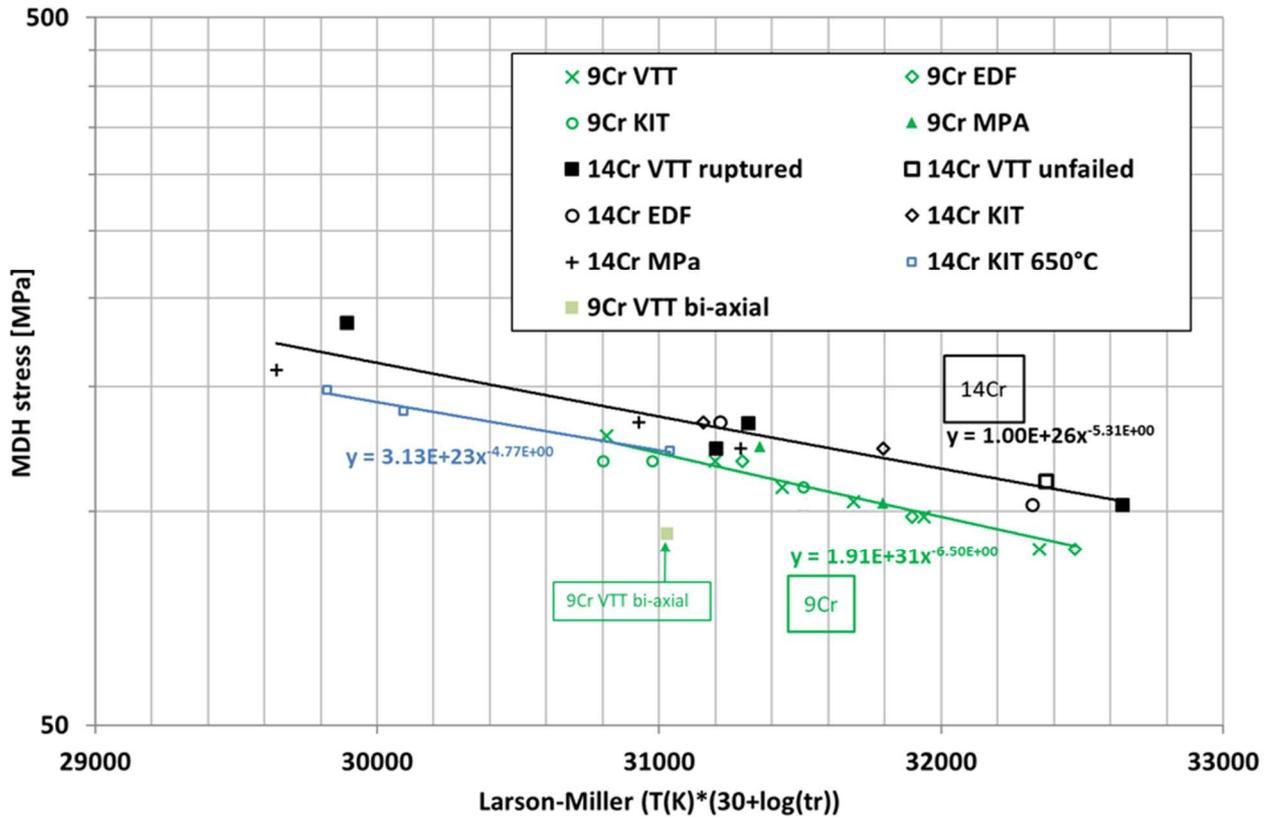
285

286 *Figure 9. Monkman-Grant plot of Internal pressure creep test at 700°C. Pressure as function of lifetime for 9Cr and 14Cr ODS*
 287 *tubes.*

288 The creep stress results are plotted as a function of the Larson-Miller parameter (with constant C=30)
 289 in Figure 10. By modifying the C-parameter of the LMP parameter, it was possible to fit the results
 290 of each material on a single line. The Mean Diameter Hoop stress is defined by equation 1, where p
 291 is the gas pressure, D is the external tube diameter and t the tube thickness.

292
$$\text{MDH Stress} = p \frac{(D - t)}{2t} \quad \text{Eq. 1}$$

293



294

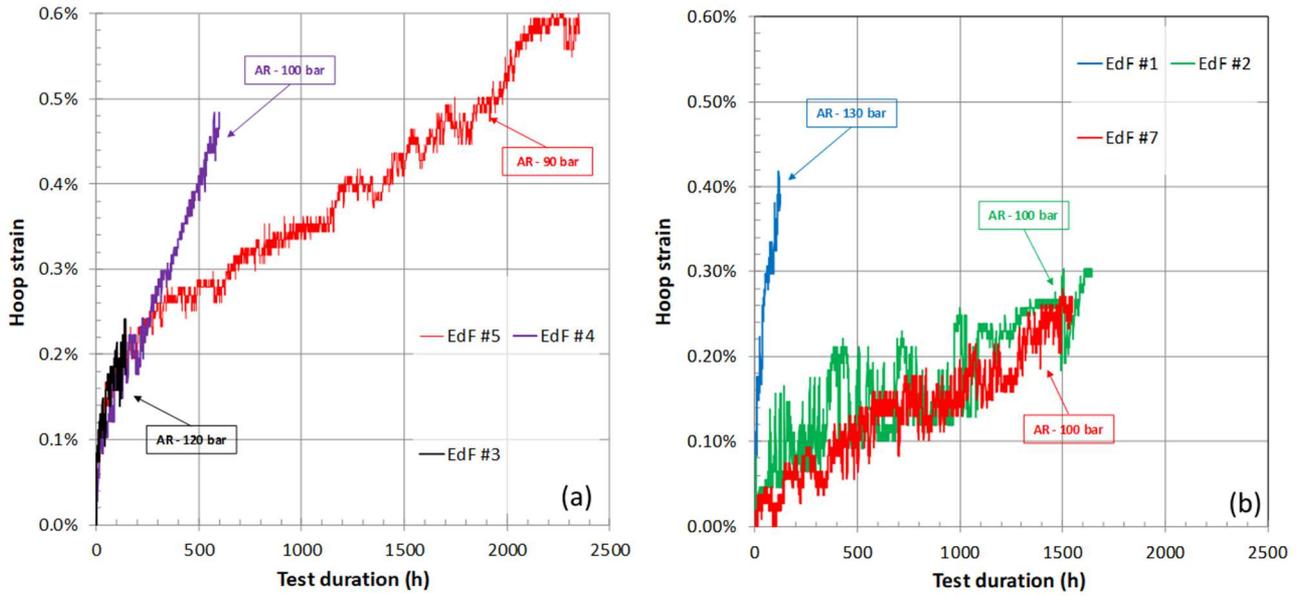
295 *Figure 10. Internal pressure tube creep test results at 700°C. Mean diameter hoop (MDH) stress as function of the Larson-Miller*
 296 *parameter.*

297

298 The 14Cr ODS tube is clearly superior to the 9Cr ODS tube in terms of lifetime. This behavior could
 299 be explained by the significant difference in nano-precipitation density between the two grades. The
 300 biaxial test using 9Cr ODS tube was successful and showed a shorter lifetime than those obtained
 301 using internal pressure only. With only one biaxial test result further conclusions on possible reasons
 302 for the shorter lifetime will not be done. The exact test conditions are summarized in the supplementary
 303 tables S1 and S2 for 14Cr ODS and 9Cr ODS tubes, respectively (Please refer to electronic
 304 supplementary material).

305 The on-line diametric measurements, i.e., the creep curves, performed by EDF, are shown in Figure
 306 11. The creep hoop strains at failure are small in both materials, but smaller in the 14Cr ODS tube
 307 material compared to the 9Cr ODS tube material.

308



309

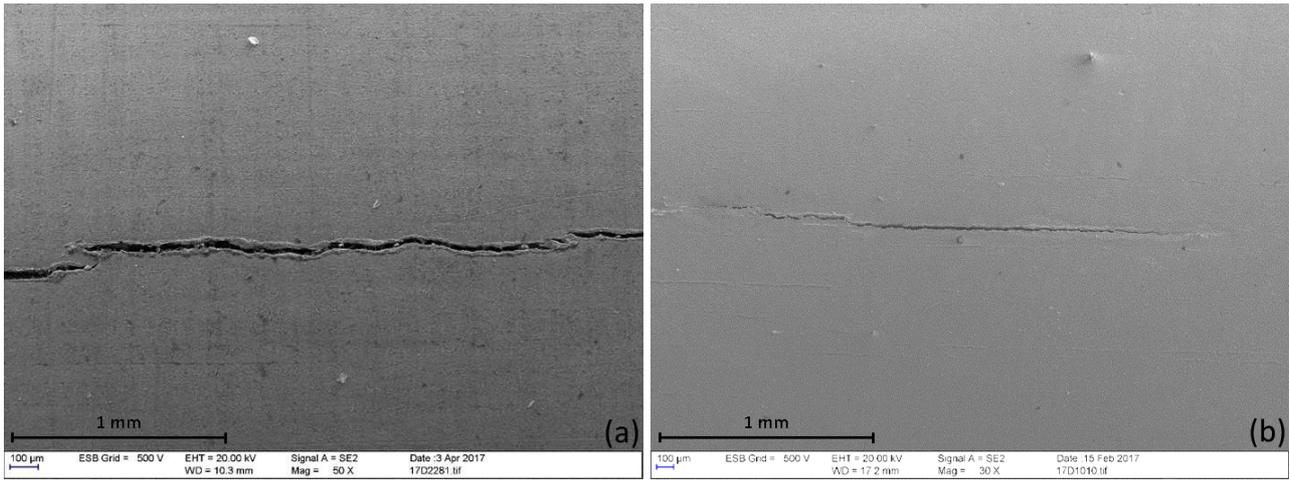
310 *Figure 11. Creep curves for (a) 9Cr and (b) 14Cr ODS tube specimens (EDF).*

311 It is clear that leakage and final failure of tube occurs without any tertiary creep stage. Minimal creep
 312 strain rate is lower for the 14 Cr ODS material compared to 9Cr ODS material at all stress levels.
 313 Even for the 9Cr ODS material, the overall hoop strain does not exceed 1 %. Considering the fuel
 314 cladding-tube application, this failure mode without burst is beneficial because it minimizes the
 315 impact of a possible burst on the coolant flow around the tube. At the same time, the very narrow
 316 crack helps in retaining the solid fuel pellets inside the tube.

317

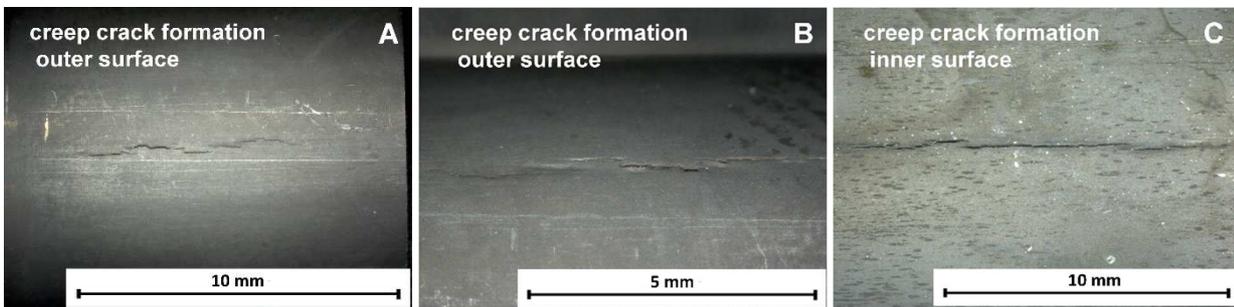
318 **3.2 Results from post-test analysis**

319 The length of the cracks after failure were typically a few mm, and they locate at least a few mm
 320 away from the plug (Figure 12 and Figure 13), strongly implying that the test results were not
 321 influenced by the plugs. Typically, the specimens contained one main crack after rupture, parallel to
 322 the rolling direction, with secondary smaller cracks near the main one. This was observed in both
 323 materials, although this feature was stronger in the 14Cr ODS material, Figure 12 (b). This behavior
 324 relates probably to the columnar microstructure of the cold rolled tubes. Previous creep studies, driven
 325 on massive samples machined perpendicular to the extrusion direction have shown a preferential
 326 crack propagation along the grain boundaries [12]. Thus, the cracks are probably prone to propagate
 327 along the grain boundaries in tubes also. The fracture surfaces were heavily oxidised, and the cracks
 328 morphology could not be determined. The oxidation is obviously occurring during the creep test. At
 329 VTT, the furnace cooling down after failure can take a few hours exposing the open crack surface to
 330 high temperature air and thus oxidation.



331

332 *Figure 12. SEM pictures of (a) 14Cr ODS tube specimen after failure (tested at 700 °C - 120 bar), (b) 9Cr ODS tube creep specimen*
 333 *after failure (tested at 700°C - 90 bar) . Several separate short cracks are seen near the main crack, causing the leak (VTT).*

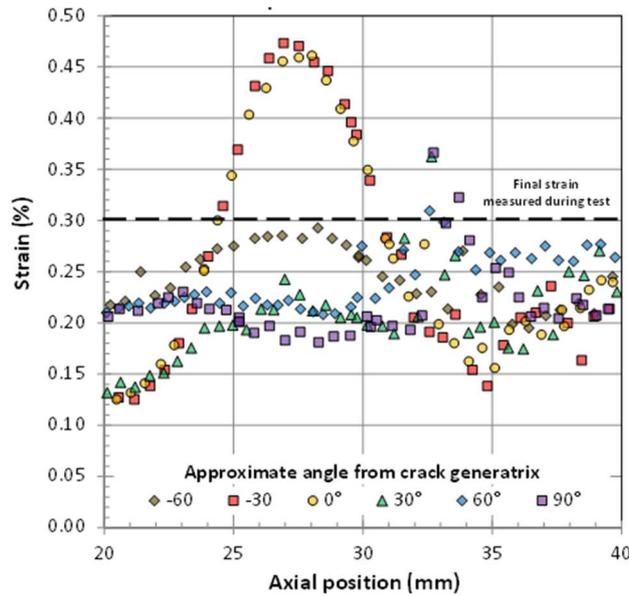


334

335 *Figure 13. Optical microscope pictures of a 14Cr ODS tube (tested at 700°C - 130 bar) - creep crack formation: (A), (B) outer*
 336 *surface, (C) inner surface (KIT).*

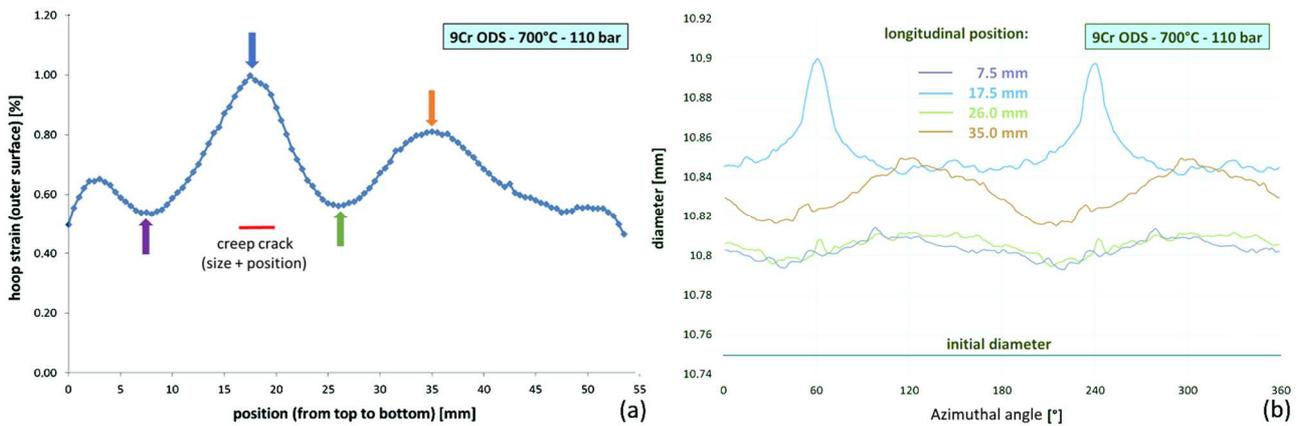
337 The main crack has almost the same length on the inner and outer sides of the tube, Figure 13, and
 338 the leak is assumed to be due to this single narrow crack.

339 Based on diametric measurements after the test, the failure strains were in all laboratories found to be
 340 smaller than 1% (mostly about 0.6 %). Figure 14 presents the measured strains in a 14Cr ODS tube,
 341 along various lines and angles. Some localisation of deformation was observed, indicating that the
 342 local failure strain is higher than the overall failure strain, and can be up to four times higher.



343

344 Figure 14. Post-test dimensional measurement for a 14Cr ODS tube specimen (tested at 700 °C-100 bar) (EDF).



345

346 Figure 15. 9Cr ODS tube creep deformations after test (tested at 700°C - 110 bar), (a) hoop strain along the tube axis and (b)
 347 Diameter along the azimuthal angle for various sections along the tube axis (KIT).

348 Figure 15 (a) presents the hoop deformation of a 9Cr ODS tube as a function of the axial position
 349 along the tube axis. For this figure, the hoop strain is computed as the mean values of the various
 350 generatrix. The diameter of the creep-tested sample is plotted as a function of various generatrix
 351 angles for different positions along the axis in Figure 15 (b). The axial position 17.5 mm,
 352 corresponding to the maximum hoop strain, exhibits two cracks localization peaks. For 9Cr ODS
 353 tubes, the average hoop strain at failure is approximately two times higher compared to the one in the
 354 14Cr ODS material. Comparison of diameter, along the crack generatrix, exhibits a more uniform
 355 creep deformation for the 9Cr ODS tube compared to the 14Cr ODS one. The 9Cr and 14Cr ODS
 356 materials respective behaviors are consistent with the noticed variation of the nano-oxides population
 357 and therefore the Zener pinning force on dislocations [32]. Indeed, the 9Cr ODS material exhibits a
 358 more ductile behavior compared to the 14Cr ODS material.

359 **4. Discussion**

360 Microstructural analysis of both 9Cr and 14Cr ODS tubes show that grain morphology is different in
 361 the two materials. The crystallographic texture is similar but more pronounced for 14Cr compared to
 362 9Cr ODS tube material. Nano-precipitate analysis both by SANS and TEM reveals finer precipitates

363 and higher density for the 14Cr ODS material. This has already been documented for other ODS
364 materials [33] and mainly attributed to the occurrence of a phase transformation at high temperature
365 in 9Cr ODS tube material.

366 Creep strength of the 14Cr ODS tube material is better than that of the 9Cr ODS tube material. This
367 is consistent with the observed differences in the nano-precipitates density that could explain a lower
368 creep strain rate in 14Cr. It can be noticed that despite a strong microstructural anisotropy in the 14Cr
369 ODS tube material, this is a good candidate for cladding tube application.

370 Previous study performs creep tests on hot extruded 14Cr ODS steel bars tested in two directions
371 [12]. Samples in the extrusion direction are obviously stronger than the ones in the transvers direction
372 in term of lifetime. Due to the dominant contribution of the hoop stress in gas pressure tests, the tubes
373 strength should be compared to the transvers direction [15]. However, the tube strengths are
374 significantly lower compared to those of the extruded bars [12]. This could be explained by the
375 nanoprecipitation coarsening induced by the intermediate high temperature heat treatments of the
376 pilgering sequence. At the same time, the material is constantly deformed along the axial direction
377 increasing the microstructural anisotropy finally observed on tubes

378 Post-test analysis of various samples shows that cracks are relatively short and propagate exclusively
379 in the longitudinal direction in both grades. This is consistent with the applied stress and the
380 microstructural anisotropy issued from cold forming. For all performed tests, leakage is due to one
381 single macroscopic crack sometime surrounded by secondary cracks in the same region. The tube
382 leakage occurs with very small radial deformation (less than 1 %) around the main crack. Monitoring
383 of tube samples geometry during creep shows that leakage occurs without evidence of tertiary creep
384 stage. This behavior is consistent with documented creep tests on forged bars [10].

385 The tests matrix analysis shows good consistency between all results, in which partners applied
386 various testing procedures, showing that the materials are homogenous, and are not affected by lab-
387 to-lab variations. The creep performance of ODS tubes strongly depends on the microstructure of the
388 material and the amount of strengthening nano-oxides. For example, with regard to martensitic ODS
389 steels, JAEA has developed tubes with a specific ferrite-martensitic microstructure with internal
390 pressure properties which are superior to those of the 9Cr ODS tubes tested in this study [24, 34].
391 Likewise, a 12Cr ferritic grade ODS with a higher W content (2wt%) than the 14Cr ODS grade
392 studied here, exhibits lifetime significantly higher than the one observed in this project [23], [24].

393 Thus, depending on the technical specification imposed by the reactor design, it appears possible to
394 increase significantly the performance of the ODS cladding tubes under internal pressure. The
395 appropriate choice of chemical composition and manufacturing route for ODS tubes will be governed
396 by the compromises between the various requirements of the intended application.

397

398 **5. Conclusions**

399 Ferritic ODS tubes were manufactured as thin cladding tubes and further tested in creep conditions
400 by inner gas pressure. The creep behavior was determined for two ODS materials with different
401 chromium content, i.e., 9Cr and 14Cr. These two materials exhibits some difference in terms of grain
402 size and shape as well as in texture and nano-precipitate distribution. The creep test matrix using
403 internal pressure was built in collaboration between EDF, VTT, KIT and MPA. The test set-ups
404 differs mainly in the experimental device and tube clamping system. The following main results are
405 obtained:

- 406
- 407
- 408
- 409
- 410
- 411
- 412
- 413
- 414
- 415
- 416
- 417
- 418
- 419
- 420
- 421
- 422
- 423
- Internal pressure creep test results obtained at 650°C and 700°C at VTT, EDF, KIT, and MPA are consistent. The same tendency are observed both for 14Cr and 9Cr ODS tube materials. All results, produced by different project partners are consistently aligned on a single Larson-miller graph despite the use of various clamping system and testing device. Thus, no lab-to-lab variation exist, and the materials are homogenous. The 14Cr ODS steel exhibits a better creep strength compared to the 9Cr ODS material. This tendency, largely documented for tensile samples from forged materials, is confirmed here for cold formed tubular materials.
 - The microstructure of 9Cr and 14Cr ODS tube materials differs strongly in terms of grain size, aspect ratio and crystallographic texture. Unexpectedly, the strong microstructural anisotropy of 14Cr ferritic grade is not detrimental for the tube creep behavior.
 - The observed differences between the 9Cr and 14Cr ODS tube creep strength ccan be linked to the differences in the nano-precipitate density and size.
 - The samples develop a single main crack of about 10 mm long that cause leakage and stop of the test. This crack induces small hoop strain near the crack, which is limited to a few percent. Secondary cracks are often observed in both 14Cr and 9Cr ODS tube materials.
 - Consistently with tensile tests, the tube creep test show no tertiary creep stage associated to a brutal cracking and leakage.

424

425 **6. Acknowledgements**

426 This work has been supported by the MatISSE European H2020 program. We kindly thanks Elodie
427 ROUESNE (CEA) for SEM-EBSD characterisation of materials and Sébastien VINCENT for his
428 contribution to creep matrix definition. The SAFIR2018 programme is acknowledged for
429 development of the Crebello biaxial testing system at VTT. At EDF, Emmanuelle SCHOENER
430 (EDF) for creep test performance, Martine BLAT (EDF) for material discussions and Frédéric
431 DELABROUILLE (EDF) for microscopic investigations are acknowledged.

432

433 **7. CRediT author statement**

434 **Denis Sornin:** Resources, Writing-original draft, investigation

435 **Ulla Ehrnstén:** Supervision, Conceptualization, Resources, summarizing results

436 **Juhani Rantala:** Methodology, Investigation Data curation

437 **Mario Walter:** Resources, Investigations

438 **Alexander Hobt:** Investigation, Methodology

439 **Lukas Frank:** Investigation, Methodology

440 **Andreas Ulbricht:** Investigation; Writing – review and editing

441 **Nathanael Mozzani:** Resources, Methodology, Investigations

442 **Solène Gicquel:** Concept, Resources

443 **Elvira Oñorbe:** Investigation, Data Curation

444 **Mercedes Hernández-Mayoral:** Investigation, Data Curation, Writing-Original Draft

445 **Yann de Carlan:** Concept, supervision

446

447 The authors declare that they have no conflict of interest.

448

449 The raw/processed data required to reproduce these findings cannot be shared at this time as the data
450 also forms part of an ongoing study

451

452 **Annex:** Tube creep matrix table for 9Cr ODS and 14Cr ODS steel

453

454 **8. References**

- 455 [1] B. Gwinner, M. Auroy, D. Mas, A. Saint-Jevin, S. Pasquier-Tilliette, *Journal of Nuclear Materials*, (2012),
456 428, 110-116.
- 457 [2] I. Proriol Serre, O. Hamdane, J.-B. Vogt, *Nuclear Engineering and Design*, (2017), 320, 17-27.
- 458 [3] P. Yvon, F. Carre, *Journal of Nuclear Materials*, (2009), 385, 217-222.
- 459 [4] T. Chen, E. Aydogan, J.G. Gigax, D. Chen, J. Wang, X. Wang, S. Ukai, F.A. Garner, L. Shao, *Journal of*
460 *Nuclear Materials*, (2015), 467, 42-49.
- 461 [5] R.L. Klueh, *Philosophical Magazine*, (2018), 98:28 2618-2636.
- 462 [6] Y. de Carlan, J.-L. Bechade, P. Dubuisson, J.-L. Seran, *Journal of Nuclear Materials*, (2009), 386-388, 430-
463 432.
- 464 [7] T. Kaito, Y. Yano, S. Ohtsuka, M. Inoue, K. Tanaka, A.E. Fedoseev, A.V. Povstyanko, A. Novoselov, *Journal*
465 *of Nuclear Science and Technology*, (2013), 50:4, 387-399.
- 466 [8] G.R. Odette, *Scripta Materialia*, (2018), 143, 142-148.
- 467 [9] S. Ukai (Ed.) *Microstructure and high temperature strength of 9Cr ODS ferritic steel.*, 2011.
- 468 [10] J. Malaplate, F. Momprou, J.L. Béchade, T. Van Den Berghe, M. Ratti, *Journal of Nuclear Materials*,
469 (2011), 417, 205-208.
- 470 [11] R.L. Klueh, P.J. Maziasz, I.S. Kim, L. Heatherly, D.T. Hoelzer, N. Hashimoto, E.A. Kenik, K. Miyahara,
471 *Journal of Nuclear Materials*, (2002), 307-311, 773-777.
- 472 [12] H. Salmon-Legagneur, S. Vincent, J. Garnier, al., *Material Sciences and Engineering*, (2018), A722, 231-
473 241.
- 474 [13] T. Tanno, Y. Yano, H. Oka, S. Ohtsuka, T. Uwaba, T. Kaito, *Nuclear Materials and Energy*, (2016), 9, 353-
475 359.
- 476 [14] Y. Li, J. Zhang, Y. Shan, W. Yan, Q. Shi, K. Yang, J. Shen, T. Nagasasa, T. Muroga, H. Yang, S. Kano, H.
477 Abe, *Journal of Nuclear Materials*, (2019), 517, 307-314.
- 478 [15] T. Jaumier, S. Vincent, L. Vincent, R. Desmorat, *Journal of Nuclear Materials*, (2019), 518, 274-286.
- 479 [16] D.L. Sornin, A. Karch, R.E. Logé, *Journal of Materials Science*, (2018), 53, 2965-2975.
- 480 [17] L. Toulbi, C. Cayron, P. Olier, R. Logé, Y. de Carlan, *Journal of Nuclear Materials*, (2013), 442, 410-416.
- 481 [18] T. Kaito, S. Ohtsuka, M. Inoue, T. Asayama, T. Uwaba, S. Mizuta, S. Ukai, T. Furukawa, C. Ito, E. Kagota,
482 R. Kitamura, T. Aoyama, T. Inoue, *Journal of Nuclear Materials*, (2009), 386-388, 294-298.
- 483 [19] P. Dubuisson, Y. de Carlan, V. Garat, M. Blat, *Journal of Nuclear Materials*, (2012), 428, 6-12.
- 484 [20] O. Doyen, B.L. Gloannec, A. Deschamps, F.D. Geuser, C. Pouvreau, A. Poulon-Quintin, *Journal of*
485 *Nuclear Materials*, (2019), 518, 326-333.
- 486 [21] G. Cao, NEUP report 10-925 (University of Wisconsin-Madison), (2013), 22.
- 487 [22] M.J. Bird, Monterey, California: Naval Postgraduate School. report 2014-6.
488 <http://hdl.handle.net/10945/42582> (2014), 91.

- 489 [23] M. Inoue, T. Kaito, S. Ohtsuka, Research and Development of Oxide Dispersion Strengthened Ferritic
490 Steels for Sodium Cooled Fast Breeder Reactor Fuels, in: V. Ghetta, D. Gorse, D. Mazière, V. Pontikis (Eds.),
491 Materials Issues for Generation IV Systems, Springer Netherlands, Dordrecht, 2008, pp. 311-325.
- 492 [24] S. Ukai, R. Miyata, S. Kasai, N. Oono, S. Hayashi, T. Azuma, R. Kayano, E. Maeda, S. Ohtsuka, Materials
493 Letters, (2017), 209, 581-584.
- 494 [25] R. Pohja, Moilanen, P., Rantala, J., Huotilainen, S., Tulkki, V., Ehrnstén, U. & Holmström, S., 2016, In:
495 Athens Journal of Sciences. , Athens Journal of Sciences, (2016), 3 227-240.
- 496 [26] C. Cayron, A. Montani, D. Venet, Y. De Carlan, Journal of Nuclear Materials, (2010), 399, 219-224.
- 497 [27] S. Ukai, R.J.M. Konings, 4.08 - Oxide Dispersion Strengthened Steels, Comprehensive Nuclear Materials,
498 Elsevier, Oxford, 2012, pp. 241-271.
- 499 [28] T. Yamashiro, S. Ukai, N. Oono, S. Ohtsuka, T. Kaito, Journal of Nuclear Materials, (2016), 472, 247-251.
- 500 [29] N. Roduit, JMicroVision. <https://jmicrovision.github.io/>, 2020.
- 501 [30] E. Oñorbe, M. Hernández-Mayoral, A. Morrison, M. Serrano, Nuclear Materials and Energy, (2019), 20,
502 100698.
- 503 [31] "Online ILL data" <https://doi.ill.fr/10.5291/ILL-DATA.1-01-146.>, 2015.
- 504 [32] J.-L. Lin, K. Mo, D. Yun, Y. Miao, X. Liu, H. Zhao, D.T. Hoelzer, J.-S. Park, J. Almer, G. Zhang, Z. Zhou, J.F.
505 Stubbins, A.M. Yacout, Journal of Nuclear Materials, (2016), 471, 289-298.
- 506 [33] M.H. Mathon, M. Perrut, S.Y. Zhong, Y. de Carlan, Journal of Nuclear Materials, (2012), 428, 147-153.
- 507 [34] T. Tanno, S. Ohtsuka, Y. Yano, T. Kaito, Y. Oba, M. Ohnuma, S. Koyama, K. Tanaka, Journal of Nuclear
508 Materials, (2013), 440, 568-574.

509