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DE LA RECHERCHE À L'INDUSTRIE





SIC/SIC COMPOSITE BEHAVIOR IN LWR CONDITIONS AND UNDER HIGH TEMPERATURE STEAM ENVIRONMENT

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TOP FUEL Reactor Fuel Performance Zurich, Switzerland

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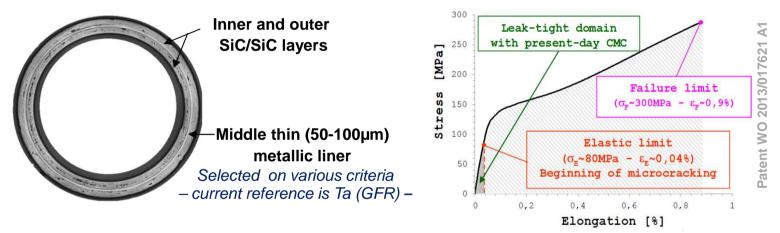
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introduction

A Decade of R&D activity at CEA has been dedicated to the development of SiC/SiC composites for GFR fuel cladding application: 2005-2015

Focus on metal/ceramic hybrid cladding: a solution to leak-tighness

CEA « SANDWICH » CLADDING DESIGN



The potential benefits in terms of application temperature and dimensional stability make this concept have a very high potential for EATF

Transposition to LWR is under investigation with the current manufacturing process

Objectives for current research (French Nuclear Institute)

- Assess the SiC resistance to corrosion/interaction with coolant in normal operating conditions
- Define the overall performance under high temperature steam



OXIDATION IN NORMAL LWR CONDITIONS

EXPERIMENTAL

0 MPa

Materials examinated

TUBULAR SIC/SIC COMPOSITES (WITHOUT LINER)

Thickness 1 mm, \varnothing 10 mm, L 75 mm

Nuclear grade materials (from CEA)

- Hi-Nicalon S fibers + CVI-SiC matrix
- Multilayered 2D architectures (filament winding, braiding with 45° angle)
- Pyrocarbon interphase between fibers and matrix

As-received and intentionally pre-damaged SiC/SiC at different level (0.05 to 0.5 % σ_R)

Acess to PyC interphase allowed to oxidizing species

HIGH PURITY CVD-SIC

 \oslash 10 mm, L60 mm

Monolithic rod samples

• For understanding and comparison to CVI-SiC matrix



- First oxidation tests in CEA autoclaves Static and closed system
 - ▶ Representative LWR nominal conditions (360°C / 180 bar) up to 3500 h 5 months
 - 2 compositions of water

Distilled water

LWR water composition

pure

1000 ppm $H_3BO_3 + 2$ ppm LiOH

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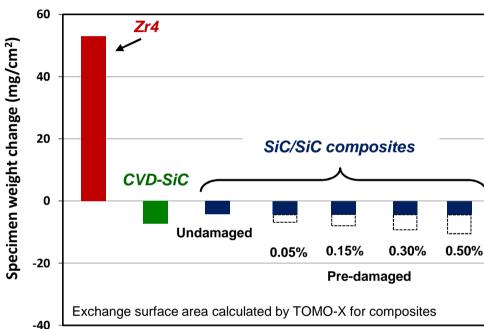
VS



OXIDATION IN NORMAL LWR CONDITIONS

WEIGHT CHANGE (LWR RESULTS PRESENTED)

After 5 months of testing



Surface crack openinas

- No visible effect of water chemistry Similar results between DW and LWR water
- Confirmation of weight loss for SiC-based samples in comparaison to weight gain for Zr

Zr4	SiC samples			
~ 3-4 µm ZrO ₂	Recession			
growth	Thickness < 0.5 µm			

Greater weight loss for pre-damaged materials in comparison to undamaged But not be quantifiable exactly due to

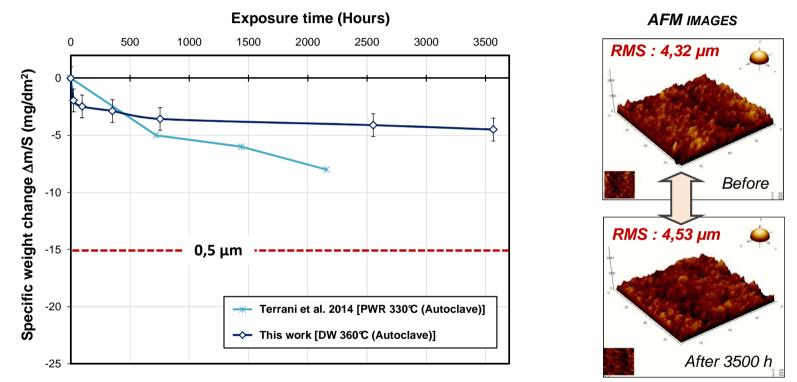
uncertinity on exchange surface

Water infiltrates through the larger crack openings and oxidizes the SiC materials from inside and not just on the surface

CORROSION RATE IS INCREASING

KINETICS AND SURFACE ROUGHNESS

Results on CVD-SiC samples (representative of the CVI-SiC matrix)

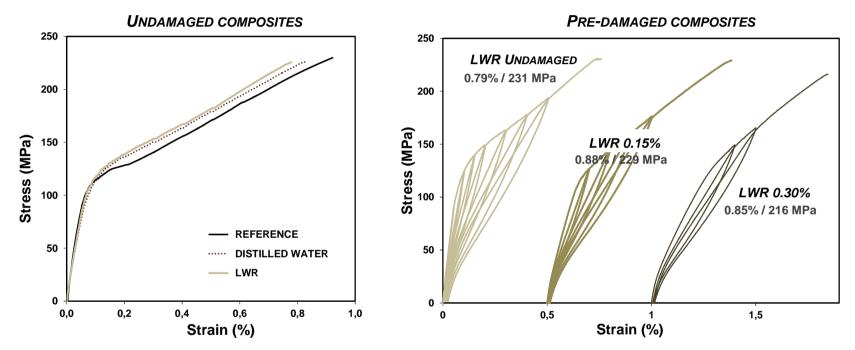


Slow kinetics of SiC recession which tends to stabilize at long time in static conditions (closed system)
 Limited effect on surface specimen (Roughness) – No passivation layer (to be confirm)

Presence of « Si » content in water after exposure suggests a release from samples

POST-EXPOSURE MECHANICAL TENSILE BEHAVIOR

No significant degradation of the mechanical behavior after long-term exposure



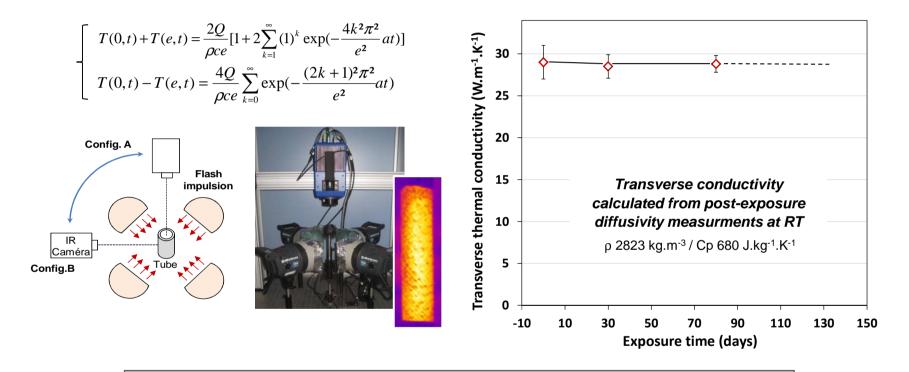
- Characteristic damageable behavior with a low fiber-matrix bonding after oxidation (DW and LWR)
- Slight reduction of strength (5%) for pre-dammaged samples in comparaison to the reference
 Pyrocarbon interphase is not significantly degraded for these oxidation conditions

Fiber-matrix load transfer remains efficient to provide ability to deform for SiC/SiC

22 OXIDATION IN NORMAL LWR CONDITIONS

POST-EXPOSURE THERMAL BEHAVIOR

Measured on undamaged SiC/SiC exposed in dynamic conditions (Loop, AREVA Facility)



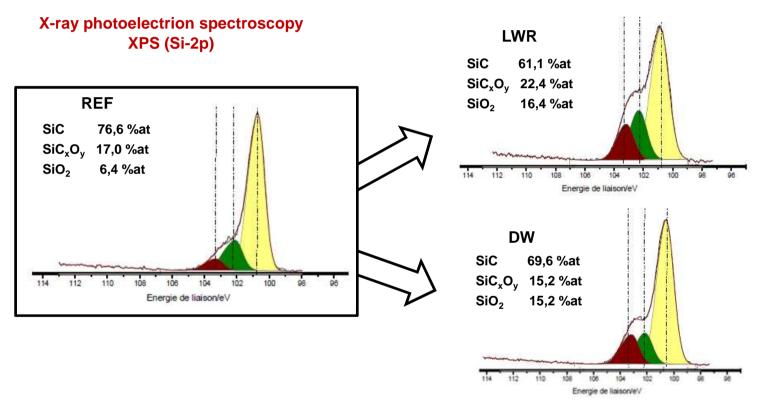
No effect of oxidation on the thermal behavior after 80 days

Consistent result with the thin sample recession and the surface analysis Next measurements after 300 et 500 days – no expected change...



DISCUSSION AND MECHANISM

Evolution of surface composition on SiC/SiC tubes before and after exposure



► After oxidation, presence of a larger (but low) quantity of SiO₂ oxide Surface is mainly composed of SiC after oxidation in both DW/LWR conditions

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OXIDATION IN NORMAL LWR CONDITIONS

DISCUSSION AND MECHANISM

- Probable mechanisms
 - 1. Direct recession of SiC

$$SiC_{(s)} + 4H_2O_{(I)} \longrightarrow Si(OH)_{4(aq)} + CH_{4(g)}$$

- 2. Formation and dissolution of silica (not protective)
 - $\begin{array}{rcl} \text{SiC}_{(\text{s})} + 2\text{H}_2\text{O}_{(\text{l})} & \longrightarrow & \text{SiO}_{2(\text{s})} + \text{CH}_{4(\text{g})} \\ \text{SiC}_{(\text{s})} + 3\text{H}_2\text{O}_{(\text{l})} & \longrightarrow & \text{SiO}_{2(\text{s})} + \text{CO}_{(\text{g})} + 2\text{H}_{2(\text{g})} \\ \text{SiO}_{2(\text{s})} + 2\text{H}_2\text{O}_{(\text{l})} & \longrightarrow & \text{Si}(\text{OH})_{4(\text{aq})} \end{array}$

In both cases, material is being consumed

Need further investigation to conclude on mechanism

(see Terrani et al., ICACC 2015 and others)

PARTIAL CONCLUSIONS:

Positive mechanical/thermal behavior of SiC/SiC in LWR conditions (Out of pile) SiC recession needs to be mitigated by protecting surface from reaction with water

Go to the next step in Assessment of performances in accidental conditions

de la recherche à l'industrie

OXIDATION OF SIC/SIC UNDER HIGH TEMPERATURE STEAM CONDITIONS





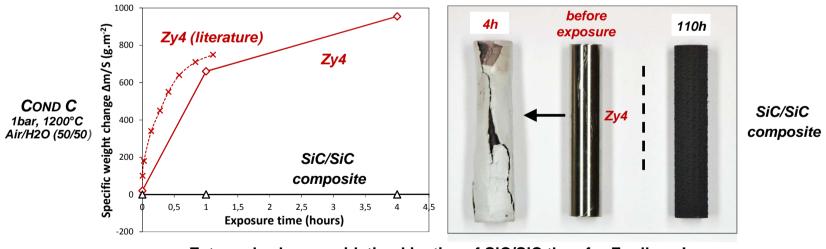
OXIDATION UNDER HIGH TEMPERATURE STEAM

EXPERIMENTAL

- **Double sided oxidation exposure for 4 to 110h**
- Steam and air/O2 flow (5cm/s) with various oxygen content Total pressure between 1 to 10 bar

Conditions		Temperature (°C)	Pressure (bar)				Pressure ratio (%)		
			Total	Water	Gas in a	ddition	Р _{н20} /Р _т	P ₀₂ /P _T	P ₀₂ /P _{H20}
HP	А	1200	10	0.5	Air	9.5	0,05 (5)	0,19 (19)	3,8
P _{atm}	В	1200	1	0.5	O ₂	0.5	0,5 (50)	0,5 (50)	1
	С	1200	1	0.5	Air	0.5	0,5 (50)	0,1 (10)	0,2
	D	1400	1	0.5	Air	0.5	0,5 (50)	0,1 (10)	0,2

Experiments on SiC/SiC tubes and Zircaloy-4 clad segments as baseline material

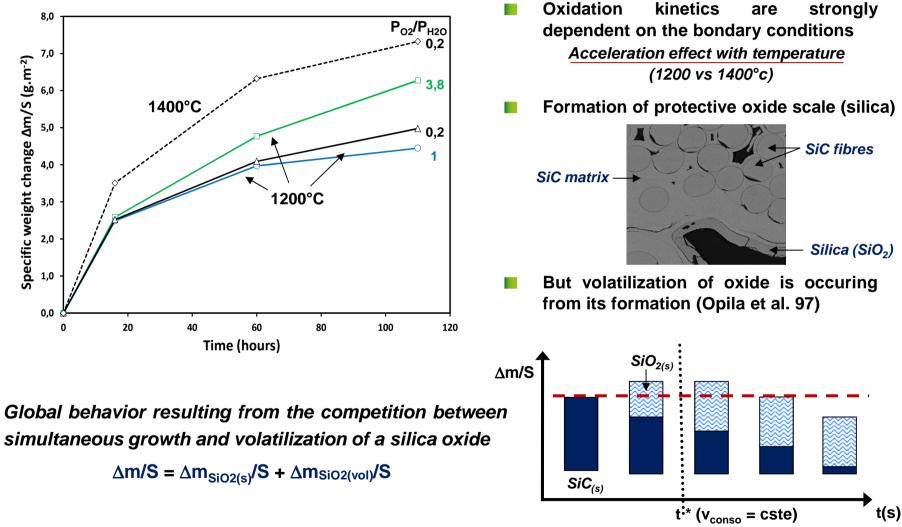


Extremely slower oxidation kinetics of SiC/SiC than for Zr alloys ! Confirmation of high potential of SiC/SiC for ATF

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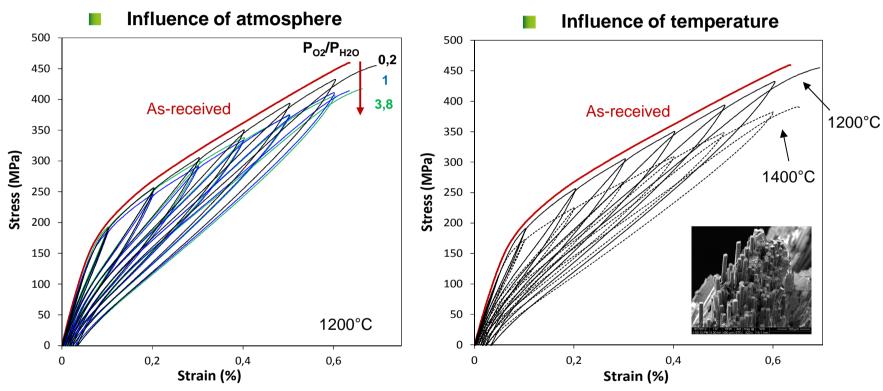
OXIDATION UNDER HIGH TEMPERATURE STEAM

FOCUS ON SIC/SIC



RESIDUAL MECHANICAL TENSILE BEHAVIOR (RT)

Integrity and geometry of specimens fully preserved after 110h exposure



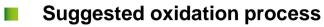
Retention of the characteristic « non-linear elastic damageable » behavior of CMCs (*No reduction of strain to failure / Slight decrease of tensile strength and Young modulus*)

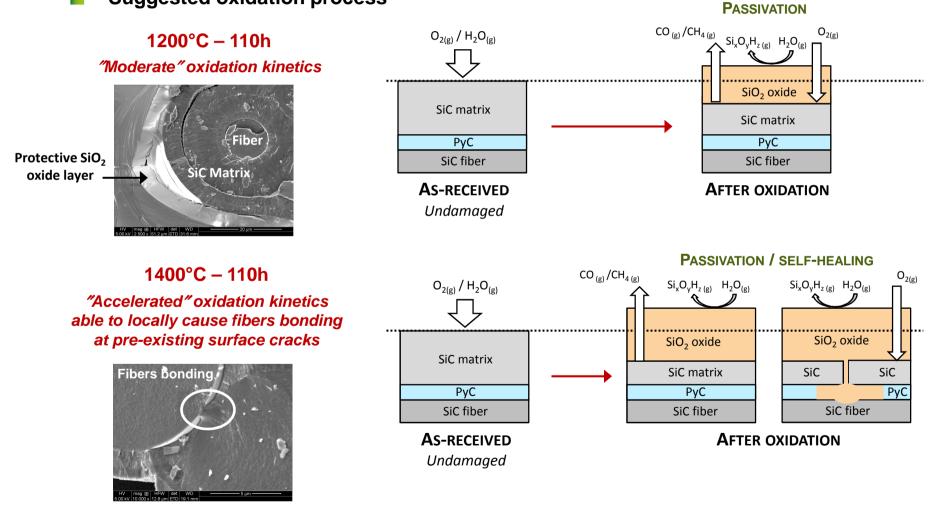
Confirmation of the protective function of silica in these conditions Pyrocarbon interphase stays efficient to deviate cracks

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OXIDATION UNDER HIGH TEMPERATURE STEAM

DISCUSSION AND MECHANISM





CONCLUSION AND PERSPECTIVES



Using SiC_f/SiC composites as fuel cladding element for LWRs represents a considerable challenge related to ambitious objectives (ATF)



SiC/SiC composites is a break-through solution which requires a long term R&D

- Technical update (present work)
 - Autoclave tests results in LWR nominal operating conditions are positive in term of mechanical and thermal behavior
 - ... but highlight a different behaviour for SiC (recession) than conventional Zr
 - High temperature strength in steam was demonstrated (up to 1400°C, > 100h)

Although many progresses have been made, many prospects have been raised to confirm the feasibility of using SiC/SiC for LWR

- On going work and some pending issues
 - > Transposition of sandwich clad to LWR requirements (design, most suitable liner)
 - Manufacturing of elongated and closed clad by welding for the « sandwich » design
 - Evaluation of neutron irradiation
 - Many others...



THANK YOU FOR YOUR ATTENTION



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