

Digital Autoradiography technique an efficient tool for sampling procedure

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DIGITAL AUTORADIOGRAPHY TECHNIQUE AN EFFICIENT TOOL FOR SAMPLING PROCEDURE

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Overview

- Context and LASE laboratory : Analytical Support to Facilities Laboratory
- Digital Autoradiography Technique
- Digital Autoradiography in support of sampling processes
- Digital Autoradiography: improvement of the selectivity
- Conclusions

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CONTEXT: MANAGEMENT OF NUCLEAR WASTES

↳ **Who?** The National Radioactive Waste Management Agency **ANDRA** is in charge of the long-term management of radioactive wastes in France

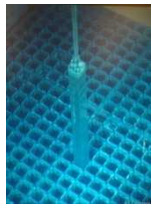


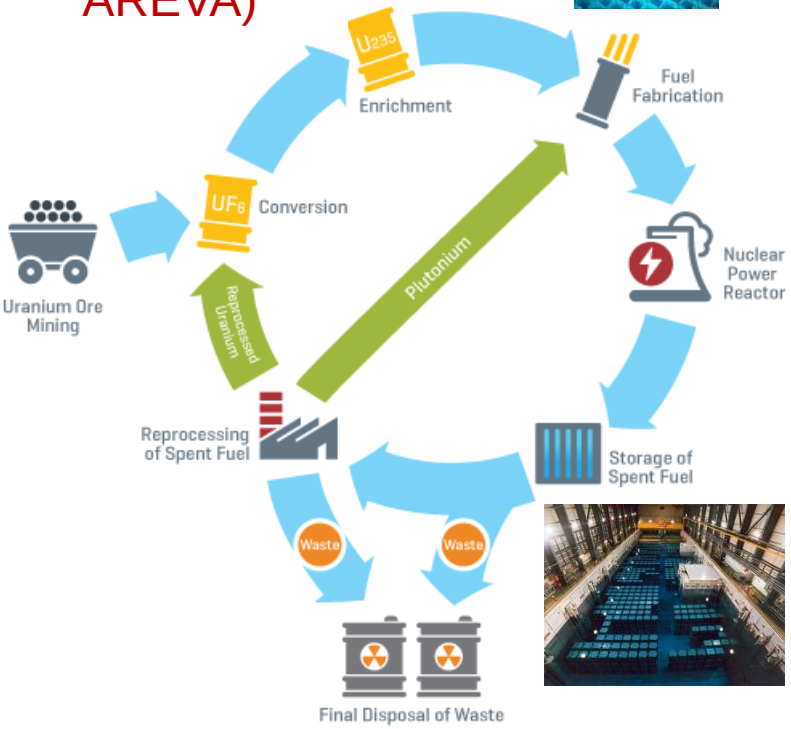
⇨ Classification of radioactive wastes as a function of their management

<table>
<tr><th>Activity - Half-life</th><th>Very short-half-life
< 100 days</th><th colspan="2">Short half-life
≤ 31 years</th><th>Long half-life
> 31 years</th></tr>
<tr><td>Very low level waste</td><td rowspan="4">Management by radioactive decay</td><td colspan="2">Surface disposal facility (CSTFA Aube facility)</td></tr>
<tr><td>Low level waste</td><td rowspan="2">Surface disposal facility (CSFMA Aube facility)</td><td>Near-surface disposal facility being studied</td></tr>
<tr><td>Intermediate level waste</td><td>Deep disposal facility at 500 meters being studied</td></tr>
<tr><td>High level waste</td><td colspan="2">Deep disposal facility at 500 meters being studied</td></tr>
</table>

CONTEXT: ORIGIN OF LOW AND INTERMEDIATE LEVEL NUCLEAR WASTES

Nuclear power industry (EDF, AREVA)



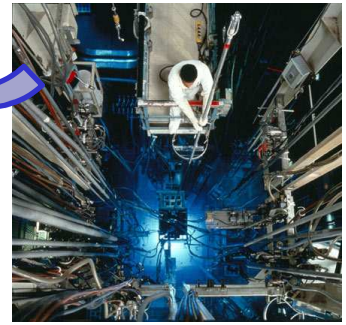


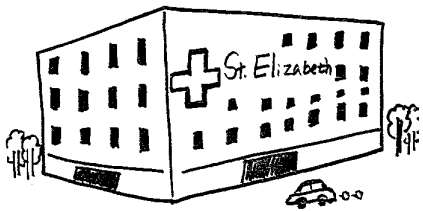










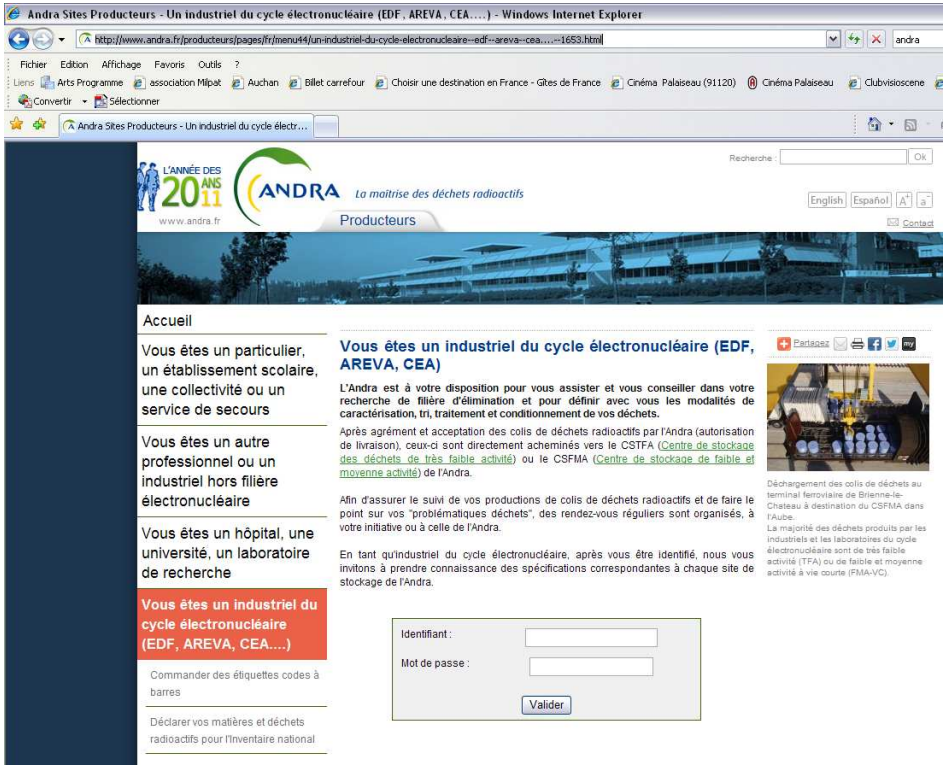


Hospitals

Nuclear research centers (CEA)

NUCLEAR WASTES ANALYSIS REQUIRED BY AUTHORITIES

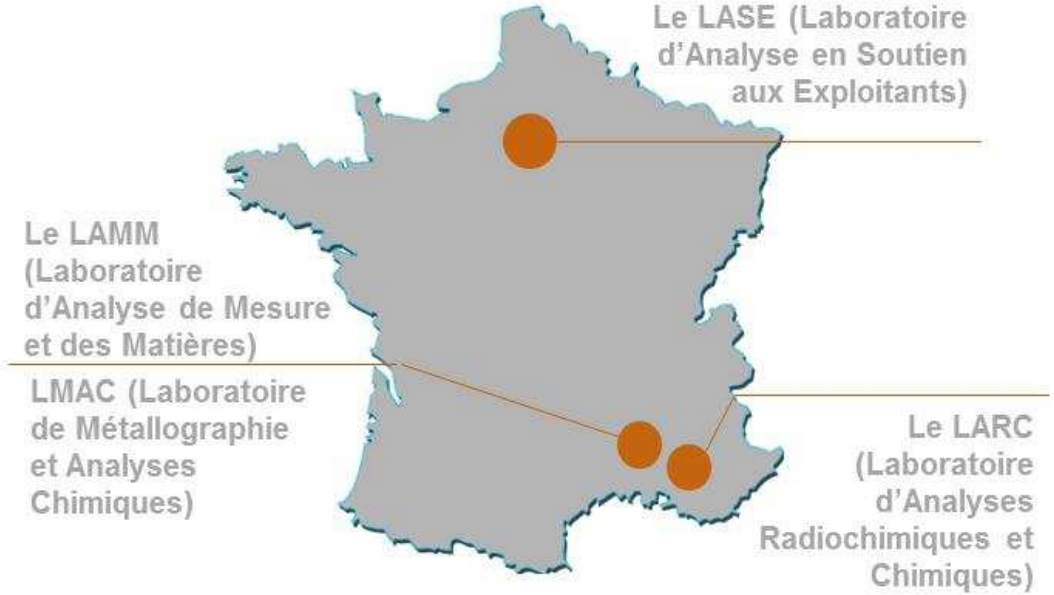
↳**Why?** **ANDRA** requests a characterization of nuclear wastes and specifies acceptance criteria for packages that waste producers have to respect



↳**Why?** **Characterization is one of the essential step in decommissioning projects**

ANALYTICAL CAPABILITY OF NETLAB

Nuclab associated to NetLab = A CEA network of analytics and experimental tools (laboratories located on CEA sites) to assist nuclear operators during characterization programs associated to dismantling process.





High Level

Intermediate level

Low level



LASE LABORATORY





- Different characterization techniques for low and intermediate level wastes.
- Destructive analysis (sample = 1 g)
- Radiochemistry
- Alpha, Gamma, LSC
- Elemental analysis

- In situ technique: Autoradiography



LASE: ANALYTICAL SUPPORT TO FACILITIES LABORATORY FRANCE

- ANALYSES ARE REQUIRED TO ASSESS THE SAFETY OF THE REPOSITORY SITES.

- LOTS OF MATRICES TO BE CONSIDERED, **Validation required**

Elemental analysis









23 fume cupboards
Radiochemistry



Nuclear analyses

CHARACTERIZATION OF LOW AND INTERMEDIATE LEVEL NUCLEAR WASTES

⇨ Mission of Analytical support to facilities Laboratory at CEA-Saclay

ANALYSIS OF ELEMENTS (RADIOACTIVE OR NOT) PRESENT AT TRACE LEVEL IN VARIOUS MATRICES

Chemical and radiological characterization of radioactive materials

Toxic elements, organic ligands, TOC, anions, cations

Radionuclides determined after radiochemistry (A~0.1Bq.g^{-1})

^{3}H, ^{14}C, ^{36}Cl, ^{55}Fe, ^{59}Ni, ^{63}Ni, ^{90}Sr, ^{93}Mo, ^{93}Zr, ^{93m}Nb, ^{94}Nb, ^{108m}Ag, ^{121m}Sn, ^{129}I, ^{151}Sm, ^{241}Pu, ^{238} et ^{240}Pu, ^{239}Pu, ^{241}Am, ^{243}Am, ^{232}U, ^{234}U, ^{235}U, ^{238}U

Wipes, technological wastes (tissues, gloves), concretes, ion exchange resins embedded in organic polymers, metals, muds, sludges, oils…









CHARACTERIZATION OF LOW AND INTERMEDIATE LEVEL NUCLEAR WASTES

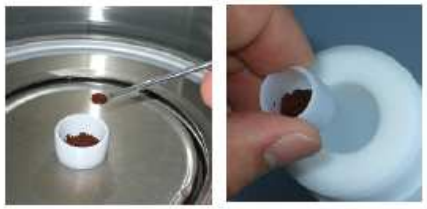
⇨ Main Radionuclides that must be investigated in priority

- Easy to measure:
  - gamma emitters
- Difficult to measure
  - Alpha emitters
  - Beta emitters
    - H-3
    - C-14
    - Cl-36
    - Sr-90

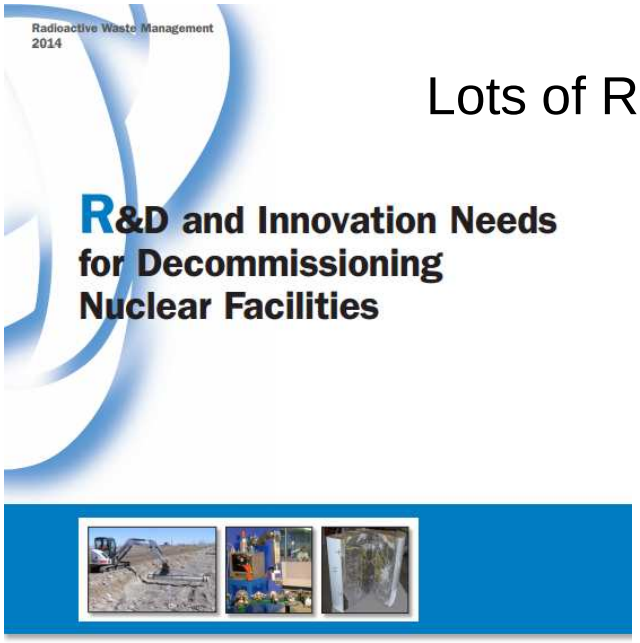
CHARACTERIZATION IS VERY IMPORTANT FOR D&D PROJECTS

Destructive techniques need less than 1 g of sample for digestion process









Lots of Radwastes must be characterized

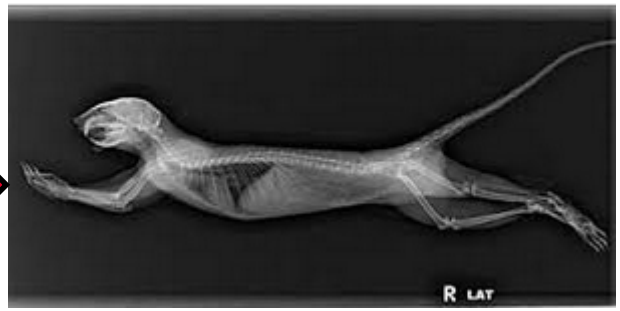


Innovation is required for in situ techniques, for techniques allowing a better sampling process

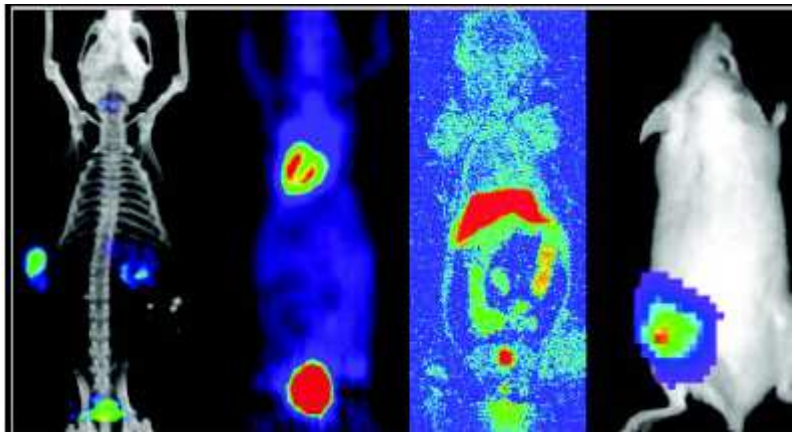
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Radiography

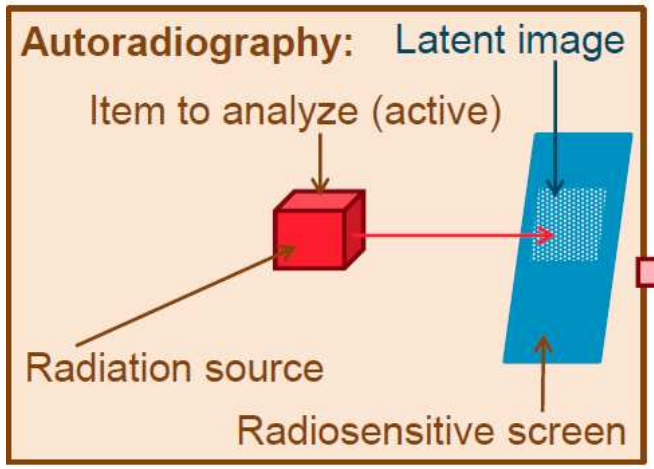
Source →  → Detector

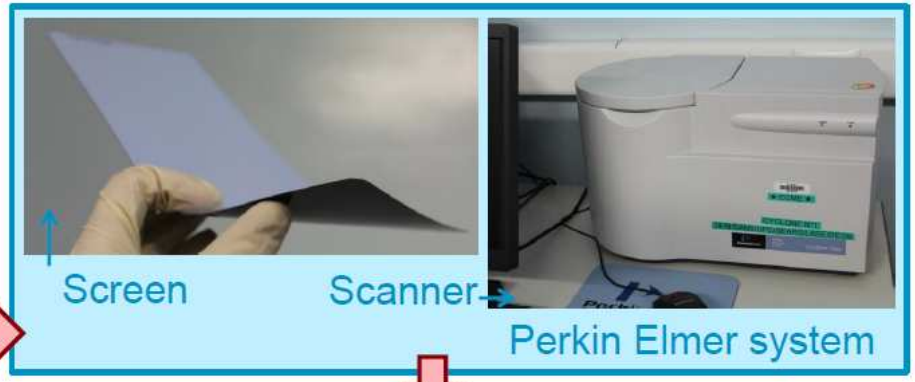
Autoradiography

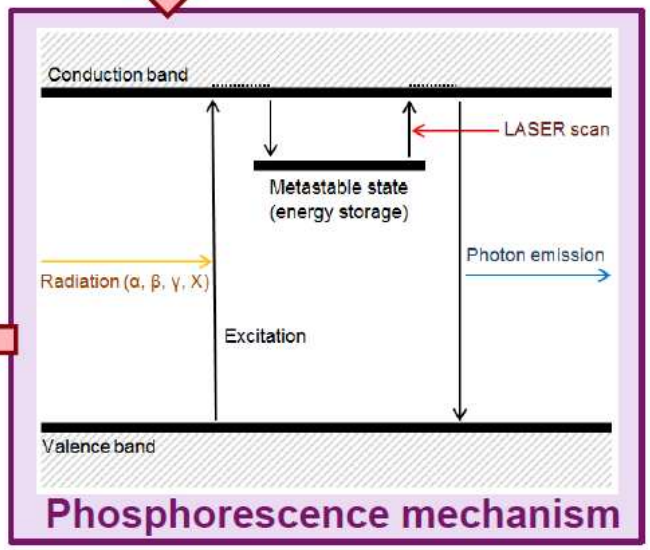
 → Detector

Interests of biologists: H, C, S, … radionuclides difficult to measure

Digital Autoradiography

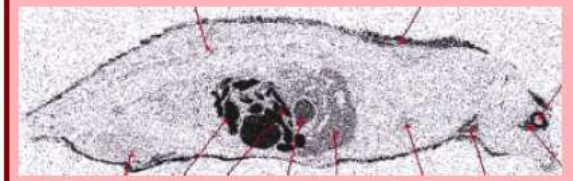








Mouse section



Whole body autoradiography

Initial application: Biomolecular research

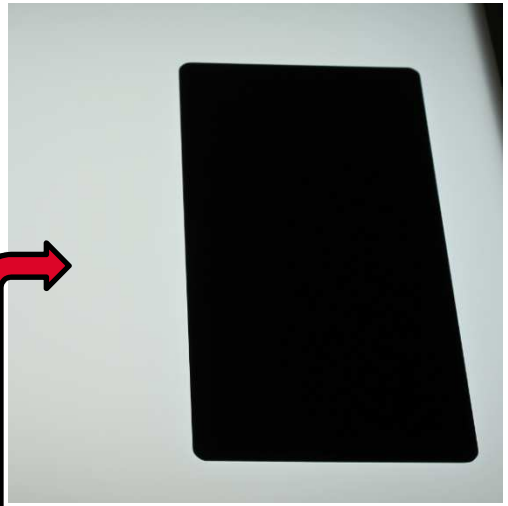
Consequences:
Small screens
High resolution
Low sensitivity

Image of radioactivity = efficient tool for sampling procedure

DA TECHNIQUE

Screens can be reused hundreds of times

(1) Screen initialization



(2) Screen Deposit



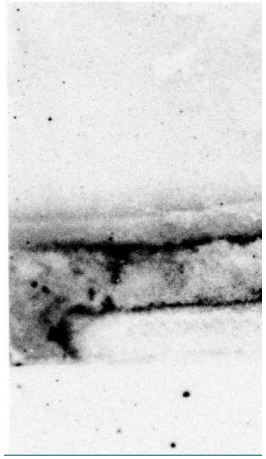
(3) Exposure Time



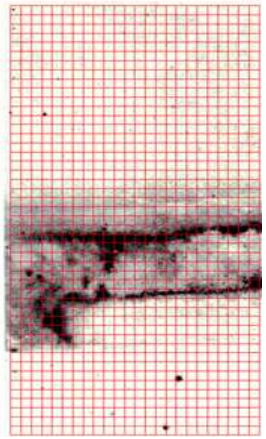
(4) Screen withdrawn



(5) Screen scan



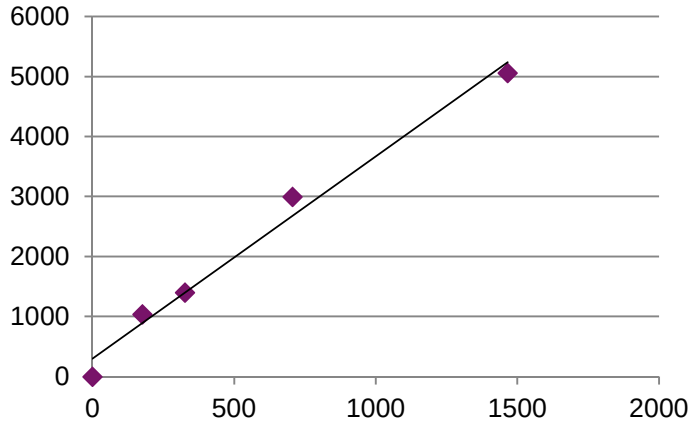
(6) Calculations



DA TECHNIQUE

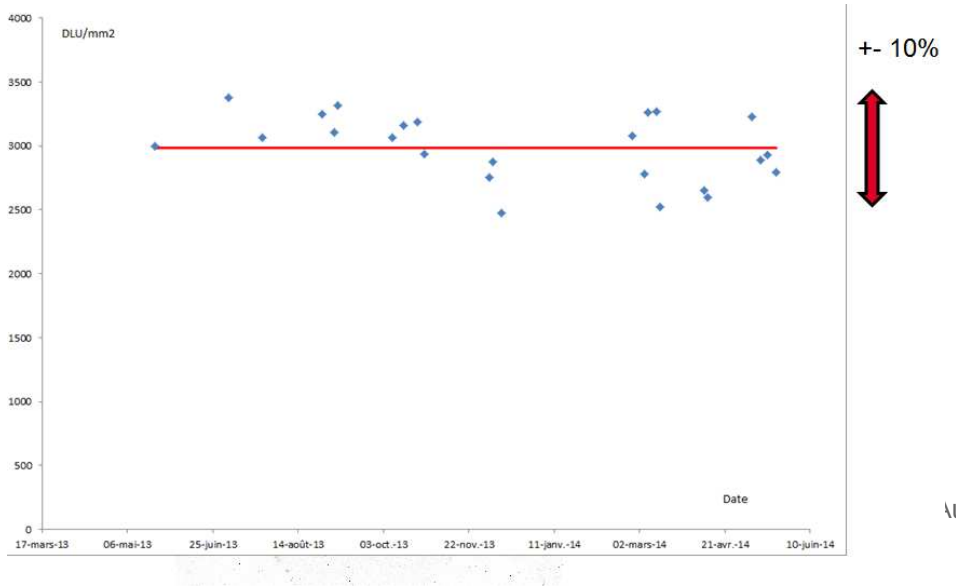
- Semi quantitative values are achievable.





- Repeatability corresponds only to a few percent

Same H-3 analysis



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SAMPLING PROBLEMS FOR RADIOCHEMICAL ANALYSIS



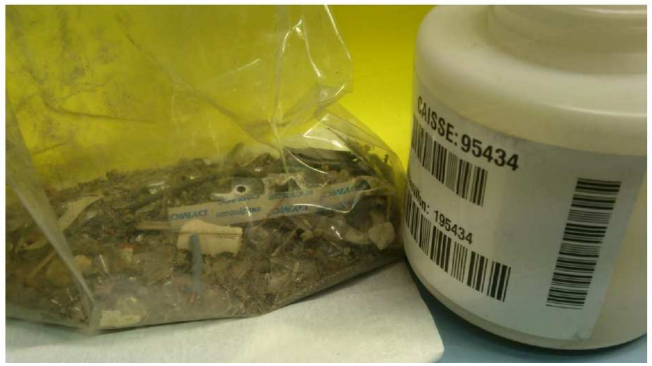
Caisse 95430



Echantillon 92886



Caisse 95434



Echantillon 95563



SAC 2 95485



How to collect 1g or even less?

DIFFERENT PROCEDURES FOR MEASUREMENTS



In Situ Sampling

Classical sampling process
by **wipe tests** = problems for non labile radioactivity

Digestion or leaching
Alpha spectrometers
Liquid Scintillation Counting
Gamma measurements
Laboratory measurements



Container

Autoradiography



In situ measurements by Autoradiography

EXAMPLE OF HOME MADE STANDARDS

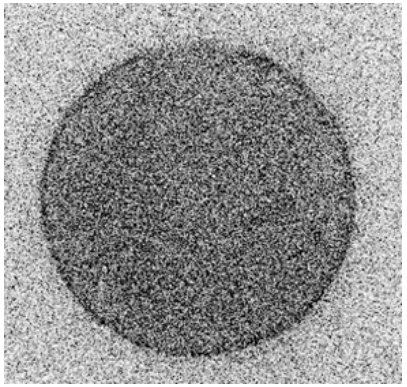
Preparation of homemade standards for beta analysis: CONCRETES

Concrete spiked at the beginning of the preparation of the cement

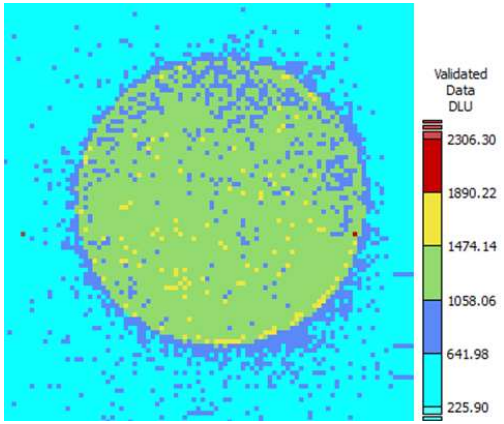
Can be spiked with numerous radionuclides H-3, C-14, Gamma emitters…)



Homogeneity on concrete surface (here H-3 and C-14)



Raw signal

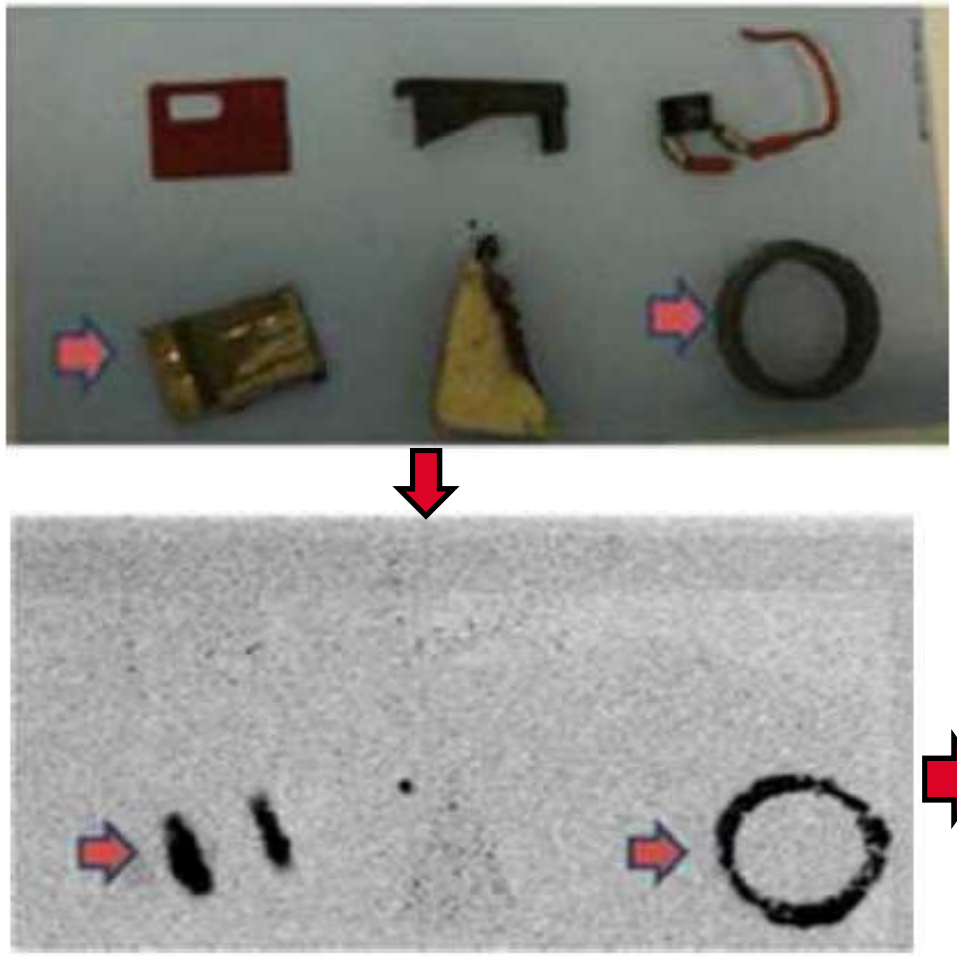


Numerical treatment to evaluate the homogeneity by digital autoradiography

EXAMPLE OF TRITIATED WASTES

Sampling process for tritiated wastes

Autoradiography



Destructive measurement of H-3 is done by pyrolysis followed by Liquid Scintillation Counting (LSC)





Sample = 1 g

H-3 AND C-14 ANALYSIS ON WASTES

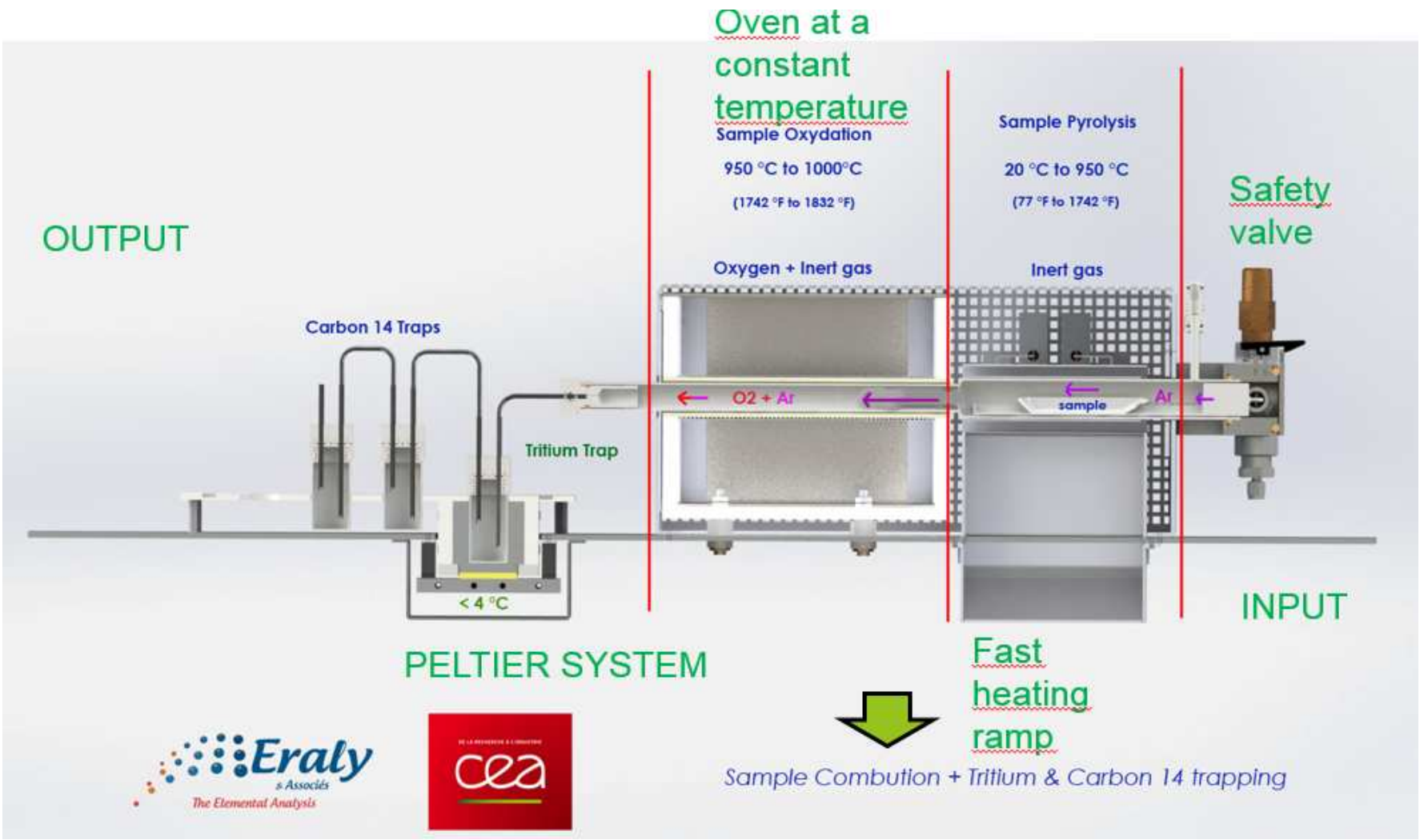
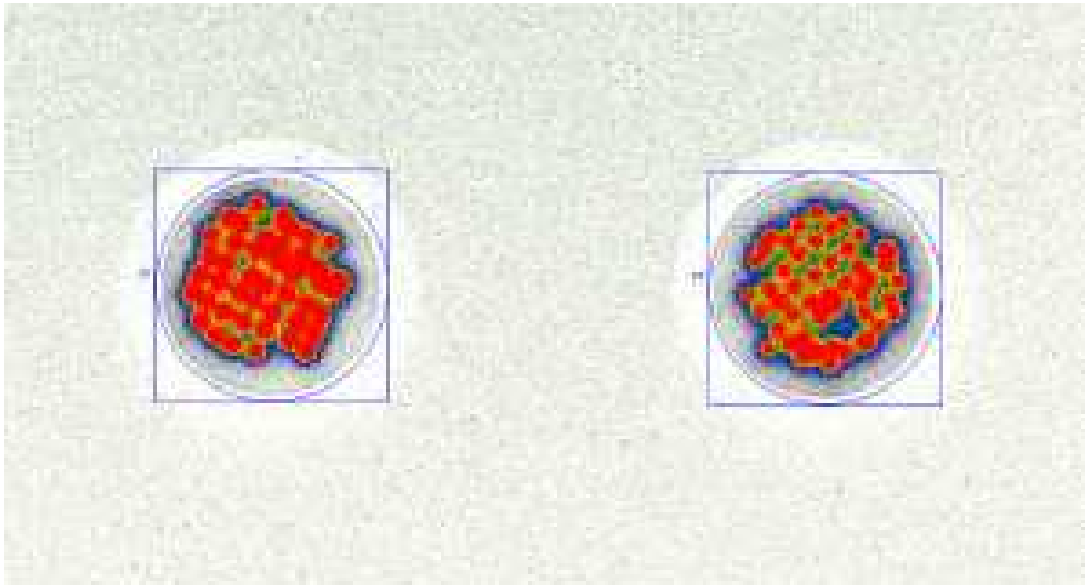


IMAGE OBTAINED WITH HIGH RESOLUTION





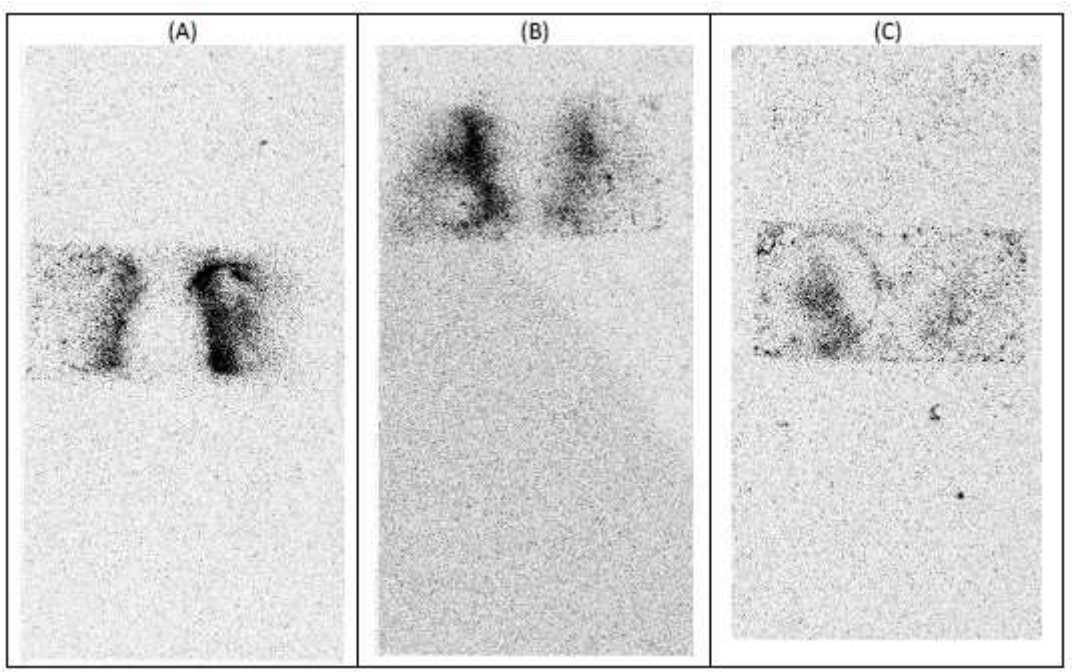
Cl-36 Sr-90

Study of sealed source by digital autoradiography – resolution used = 40 µm

EXAMPLE OF WASTES CONTAINING URANIUM

Only 3 wipes among 7 contained Uranium.





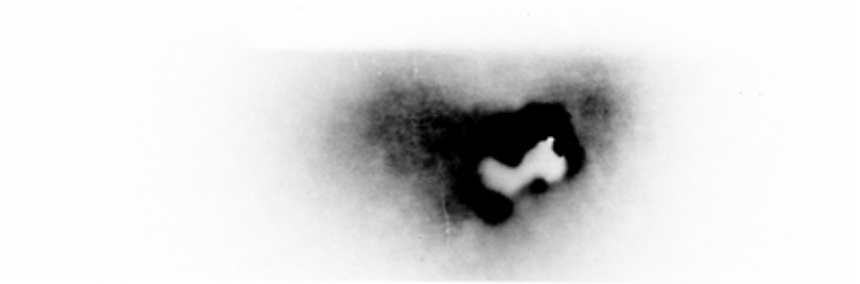
After studies with a destructive method: wipe digestion followed by alpha spectrometry and/or ICP-MS, detection limit was determined at **0.2 Bq/wipe** for Uranium (more sensitive than alpha spectroscopy).

HOW TO COLLECT SAMPLES ON CORES

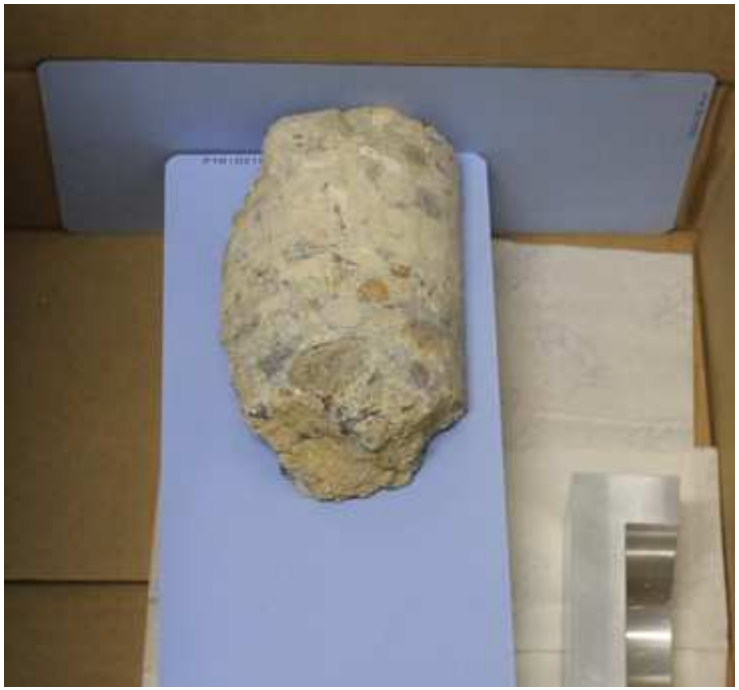
After surface analysis on D&D sites, determination of 3D contamination



Core made of concrete containing C-14



Activity on surface



Interesting sampling



Activity in depth

Image in real scale

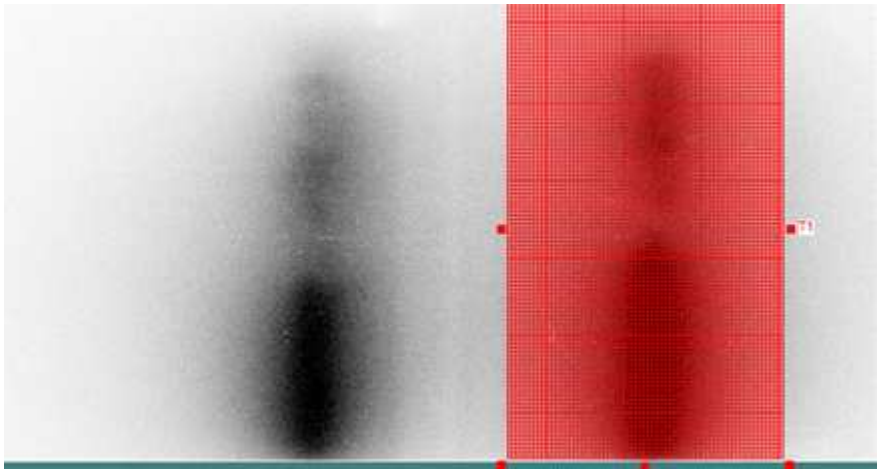
HOW TO COLLECT SAMPLES ON CORES WITH HIGH AMOUNT OF RADIOACTIVITY ?

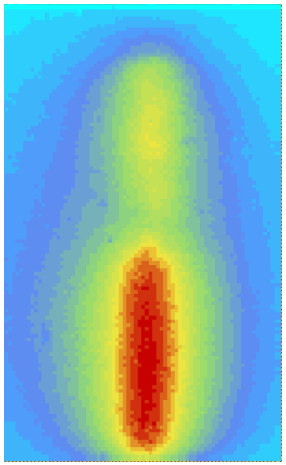
Study of alumina beads containing I-129 located inside a plastic tube





Plastic deposited on a film for **15 minutes**





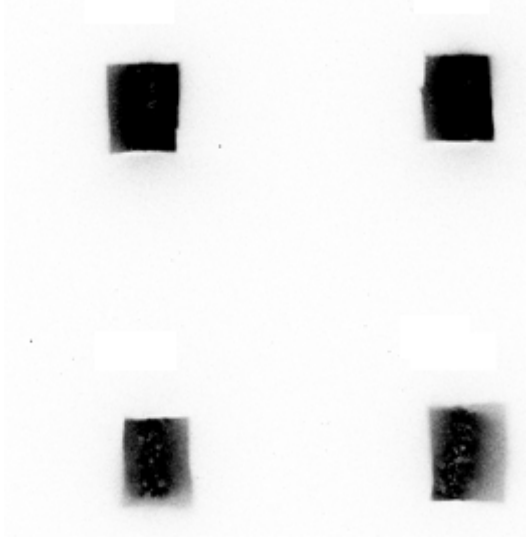
DA as efficient solution for sampling process

SAMPLING PROBLEM ON GRAPHITES

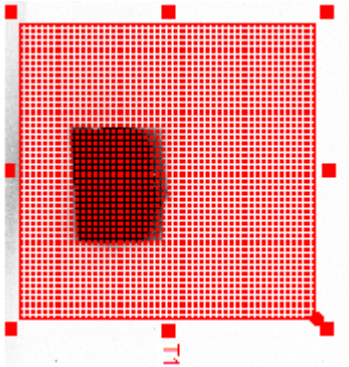
4 small cores on the same screen

Graphite cores mainly containing C-14

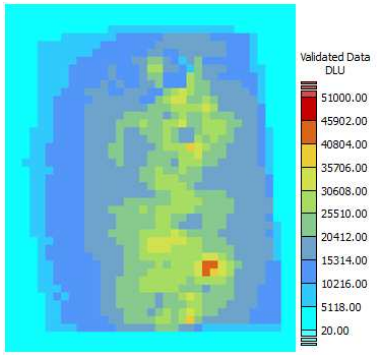




Calculations



Homogeneity of the contamination at a very low scale



Digital Autoradiography is a non destructive method to assess the homogeneity

SAMPLING PROBLEM ON STEEL MATERIALS



Contamination location and homogeneity

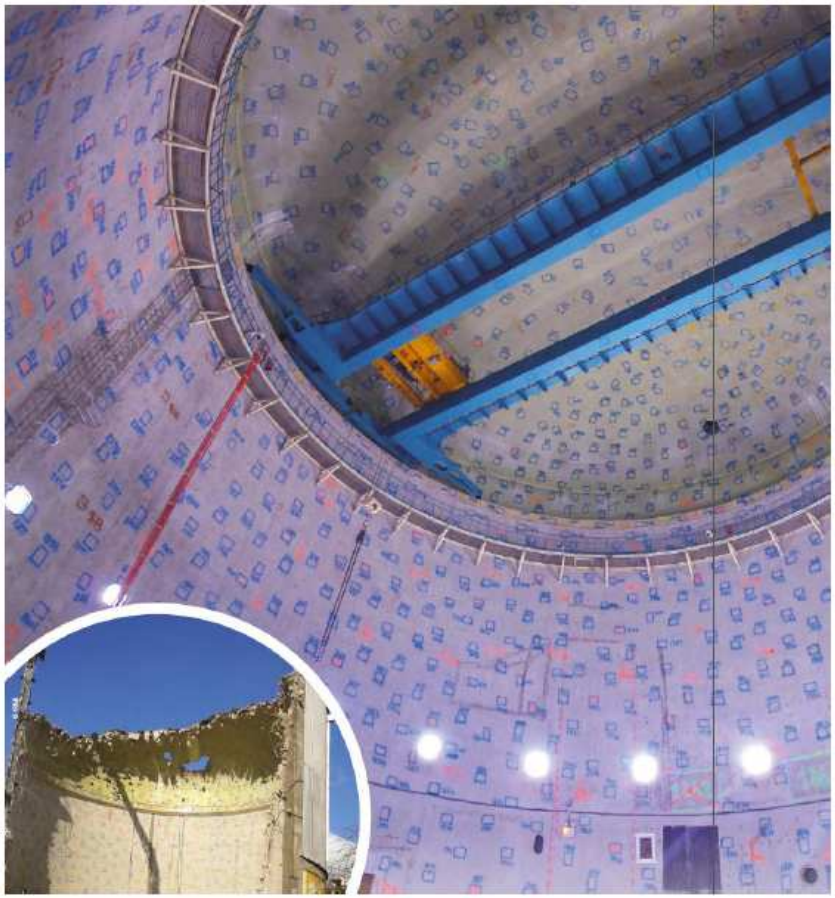
Massive steel samples investigated by digital autoradiography

For some samples only one side is contaminated

SAMPLING FOR FINAL CONTROL TO ASSESS THE CLEARANCE LEVEL

Decommissioning requires characterization at very low level. Requirement of in situ technique to improve sampling process. 1 g sample needed for destructive analysis.

Decommissioning and dismantling



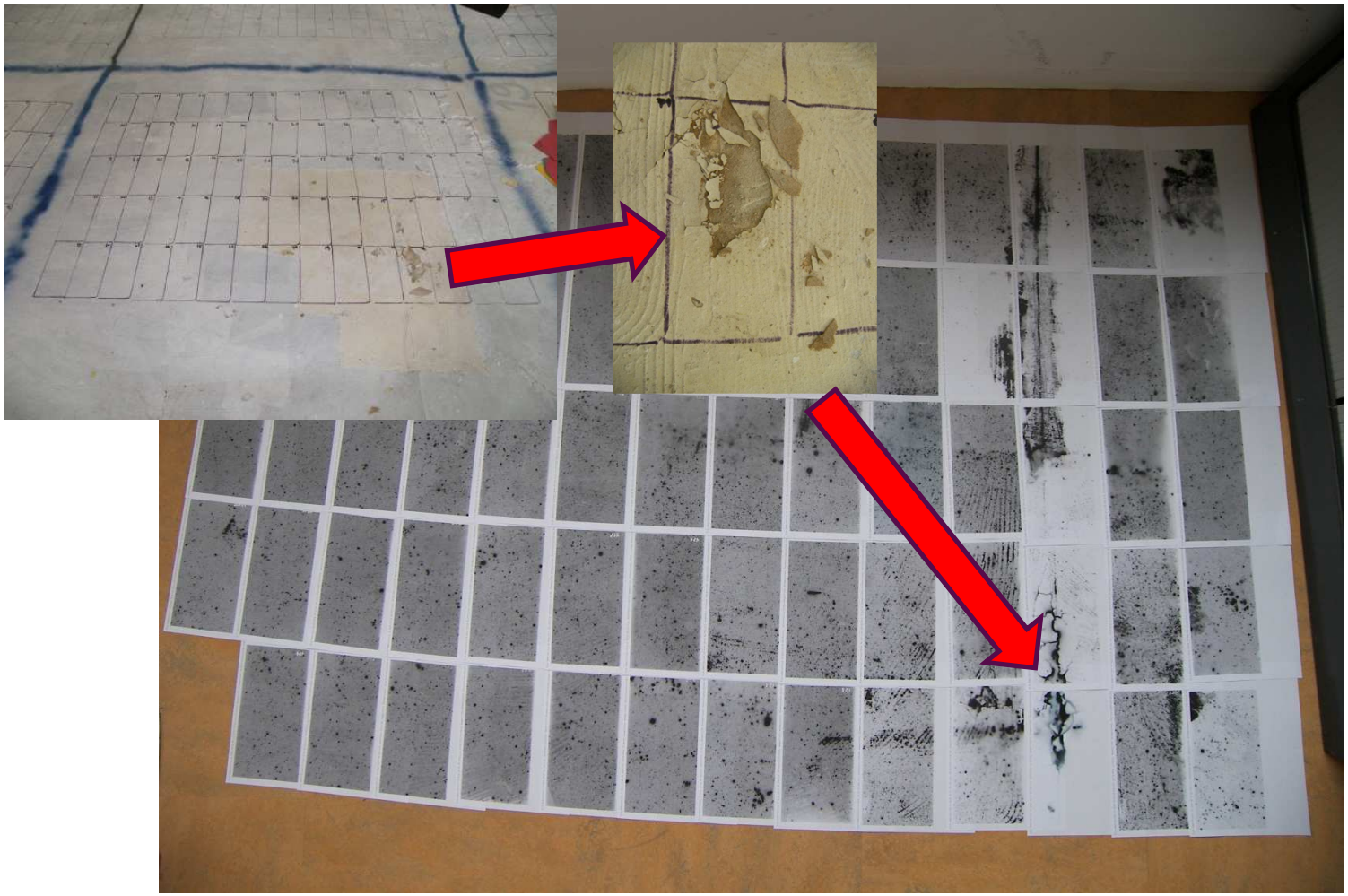
Mélusine, Grenoble





Sampling process on floor

A grid corresponding to 70 screens = 5 m^2



Radioactivity image, here C-14
Localisation of the sample for destructive analysis is clear

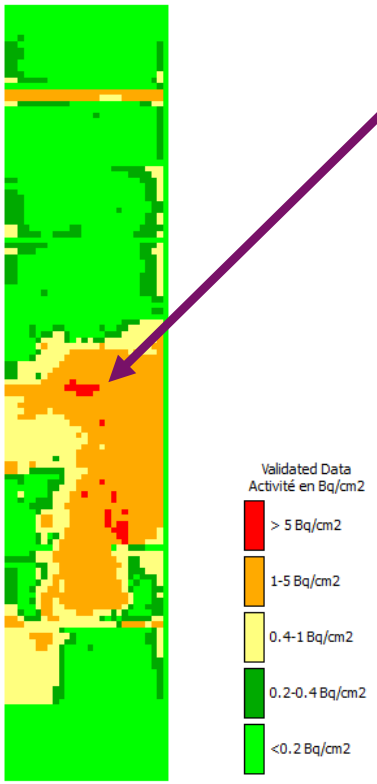
Autoradiography with raw images

OTHER EXAMPLE FOR URANIUM MEASUREMENTS



Screen on surface

1.8 m

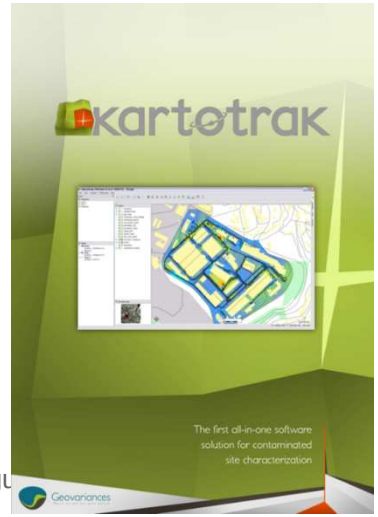


0.3 m

« Hot » spots clearly appears

Maximum activity found around 6 Bq/cm^2

Contamination shape is also an important parameter for stakeholders.



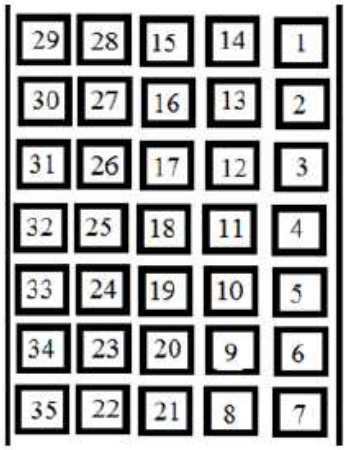
OTHER EXAMPLE FOR TRITIUM SAMPLING

Current development: geostatistical approach

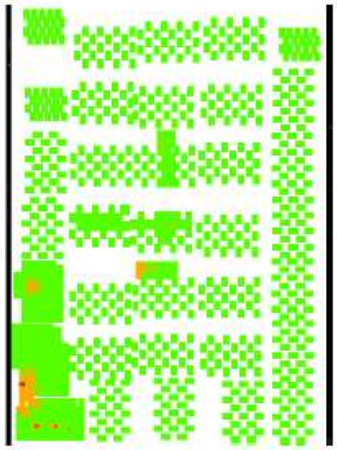


Current investigation rate:
2 weeks / 100 m²

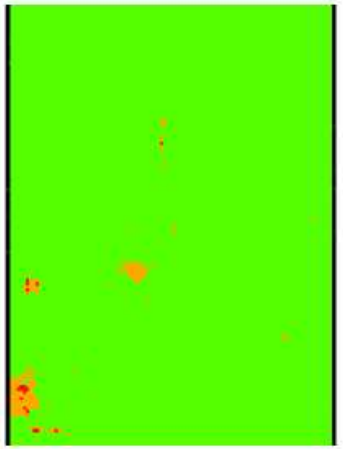
Conclusion:
Contamination accurate location
Representative and limited sampling enabling



Zoning

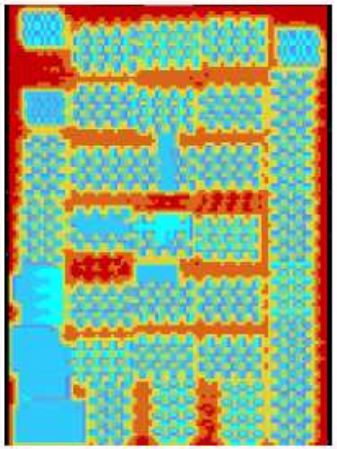


Effective mapping



Geostatistical mapping

+



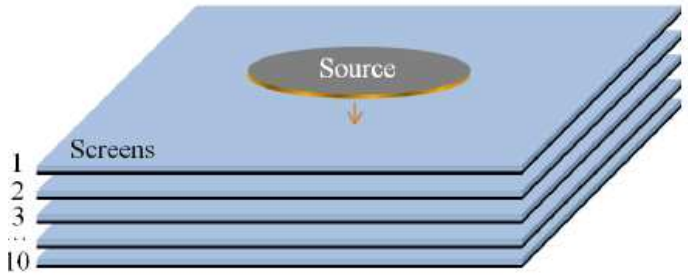
Variance plot

Overview

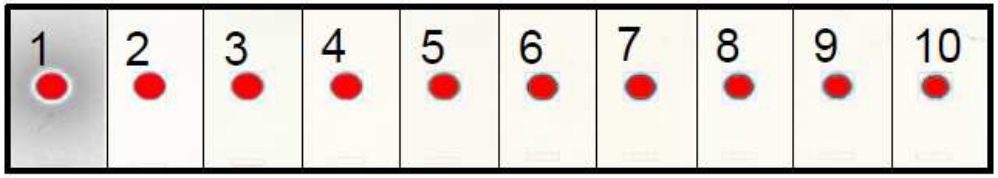
- Context and LASE laboratory : Analytical Support to Facilities Laboratory
- Digital Autoradiography Technique
- Digital Autoradiography in support of sampling processes
- Digital Autoradiography: improvement of the selectivity
- Conclusions

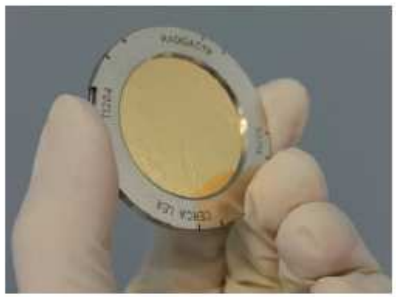
IMPROVEMENT OF SELECTIVITY

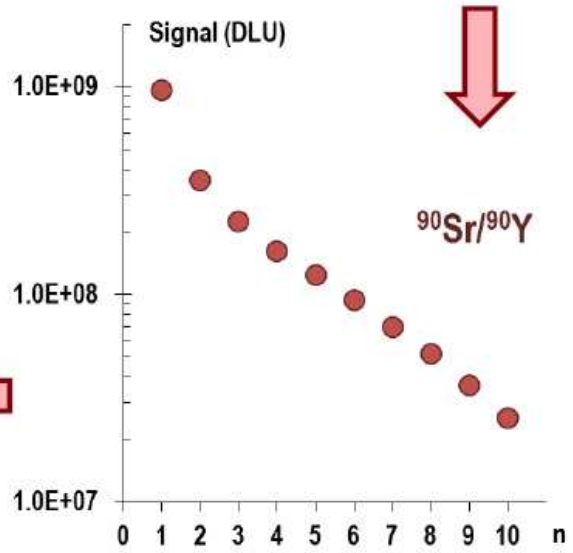
Screen stacking method (1/2)

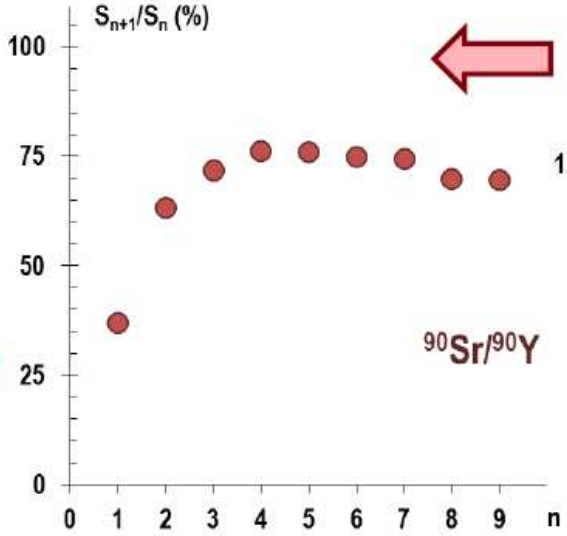


$^{90}Sr/^{90}Y$ sealed source









Intermediate precision

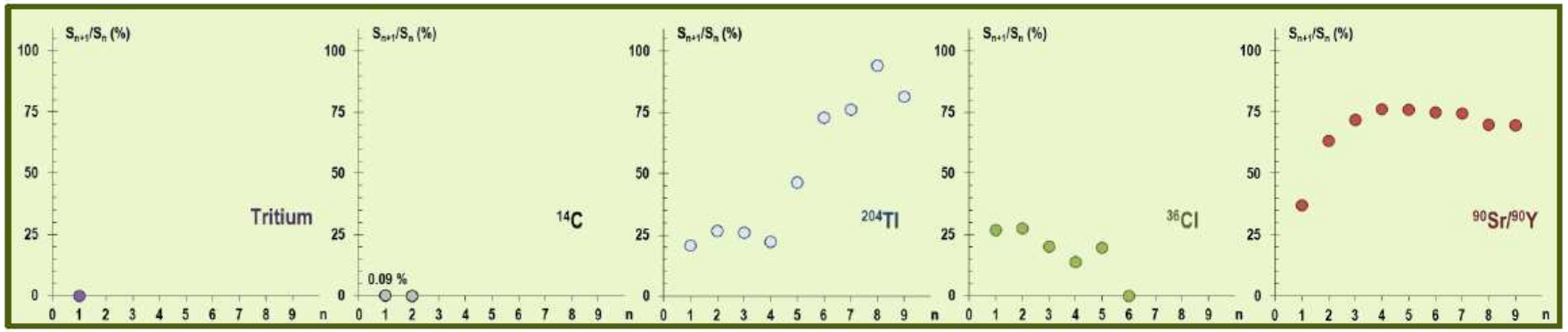
example of $^{90}Sr/^{90}Y$:

2 point sealed sources from LASE
5 sealed filtrates from LASE
1 point sealed source from INSTN
2 area sources from INSTN

S_2/S_1 = 33±6 %
S_3/S_2 = 59±6 %

IMPROVEMENT OF SELECTIVITY

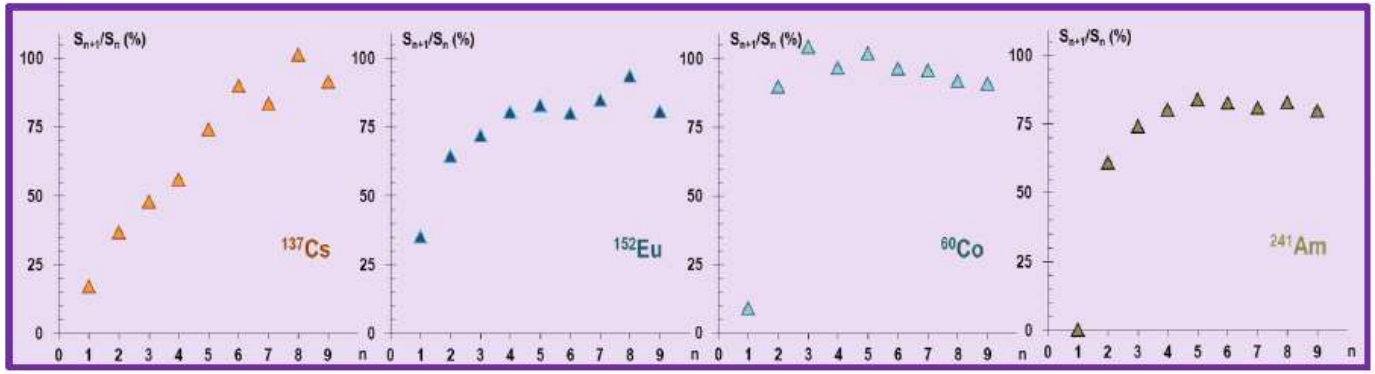
Screen stacking method (2/2)

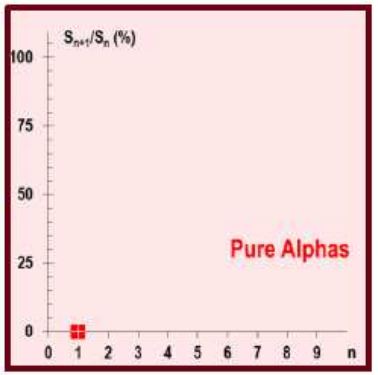


β

Radionuclides “signatures”

β+γ, α+X





α

Overview

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CONCLUSIONS

- Digital Autoradiography was used first because the technique is sensitive to difficult to measure radionuclides.
- Autoradiography appears to be a very interesting technique for sampling processes.
- Different developments are currently in R&D for mapping applications, to improve the sensitivity, to try to find possibilities to improve the selectivity.
- The LASE laboratory participates in numerous intercomparison tests where no sampling problem is usually encountered however it is totally different for radiochemical analysis required for radwastes matrices.

REFERENCES

References:

1) «Tritium analysis in building dismantling process using digital autoradiography » P Fichet, F Bresson, A Leskinen, F Goutelard, J Ikonen, M Siitari-Kaupi, Journal of Radioanalytical and Nuclear Chemistry, 291: 869-875 (2012).
2) « Digital autoradiography (DA) in quantification of trace level beta emitters on concrete » A Leskinen, P Fichet, M Siitari-Kauppi, Fl Goutelard, Journal of Radioanalytical and Nuclear Chemistry 298 : 153-161 (2013).
3) « A non-destructive and on-site digital autoradiography-based tool to identify contaminating radionuclide in nuclear wastes and facilities to be dismantled» R Haudebourg, P Fichet **:** J Radioanal Nucl Chem to be published 2016

Oral presentations:

1) «Radiological mapping of facilities under dismantling process by Digital Autoradiography», Waste Management Phoenix 2015.
2) «Digital Autoradiography : a novel complementary technique for the investigation of radioactive contamination in nuclear facilities under dismantlement» ANNIMA 2015, Lisbon, R Haudebourg, P Fichet, F Goutelard.
3) « Radiological mapping of nuclear facilities under dismantlement drawn up by Digital Autoradiography technique» R. Haudebourg, P. Fichet, F. Goutelard, Global 2015 Paris

Thank you for your attention



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